

STUDY OF THE EIGENVALUE SPECTRA OF THE NEUTRON TRANSPORT PROBLEM IN PN APPROXIMATION

*Original*

STUDY OF THE EIGENVALUE SPECTRA OF THE NEUTRON TRANSPORT PROBLEM IN PN APPROXIMATION / Saracco, Paolo; Abrate, Nicolo; Burrone, Miriam; Dulla, Sandra; Ravetto, Piero. - ELETTRONICO. - (2020). (Intervento presentato al convegno International Conference PHYSOR 2020 tenutosi a Cambridge).

*Availability:*

This version is available at: 11583/2841543 since: 2020-10-16T11:17:58Z

*Publisher:*

American Nuclear Society

*Published*

DOI:

*Terms of use:*

This article is made available under terms and conditions as specified in the corresponding bibliographic description in the repository

*Publisher copyright*

(Article begins on next page)



In this work, different formulations of the eigenvalue problems are compared in terms of characteristics of the resulting model and physical significance. The numerical solution is then achieved adopting a  $P_N$  approximation for the angular variable, focusing on some specific features of the solution when comparing odd- and even-order expansions [5]. The solution approach adopted, applied to simplified configurations, allows to calculate a set of eigenvalues, thus accessing the information related to higher-order harmonics. The eigenvalue spectra for the different formulations of the problem will be compared and discussed.

## 2. Eigenvalue problem formulations

According to what is the physical phenomenon of interest for a certain system, the eigenvalue problem associated to the transport (or diffusion) equation,

$$\hat{T}\vec{\varphi} + \hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} = \hat{S}\vec{\varphi} + \hat{F}\vec{\varphi}, \quad (1)$$

where  $\hat{T}$  is the time operator,  $\hat{L}$  is the leakage,  $\hat{R}$  is the removal,  $\hat{S}$  is the scattering,  $\hat{F}$  is the total fission and  $\vec{\varphi}$  is the flux, can be formulated in different ways, each one providing a different interpretation of the problem [6].

The most popular approach to find the steady state neutron distribution is the one traditionally introduced by Fermi, i.e. the criticality eigenproblem, where the  $\kappa$  eigenvalue is introduced in the steady state balance equation as a "tuning" parameter for the fission operator,

$$\hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} - \hat{S}\vec{\varphi} = \frac{1}{\kappa}\hat{F}\vec{\varphi}. \quad (2)$$

A similar approach for the stationary equation focuses on the collisions, dividing both the scattering and the fission operators by the  $\gamma$  eigenvalue,

$$\hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} = \frac{1}{\gamma}(\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi}). \quad (3)$$

A more exotic and less explored alternative to study the steady state chain reaction is to look for the material density allowing the system to become critical: this objective is achieved through the introduction of a  $\delta$  eigenvalue for all collision terms in the balance equation:

$$\hat{L}\vec{\varphi} = \frac{1}{\delta}(\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi} - \hat{R}\vec{\varphi}). \quad (4)$$

As far as the time-dependent problem is concerned, assuming suitable initial conditions, it is possible to substitute the time operator  $\hat{T}$  with the so-called *time eigenvalue*,  $\alpha$ , which has the physical meaning of the inverse of the reactor period:

$$\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi} - \hat{R}\vec{\varphi} - \hat{L}\vec{\varphi} = \frac{\alpha}{v}\vec{\varphi}. \quad (5)$$

Such formulation, which can also be interpreted as a Laplace transform of the time-dependent balance model, is more correctly representing the asymptotic behavior of a nuclear system when delayed fission emissions are included, leading to:

$$\begin{cases} \frac{\alpha}{v}\vec{\varphi} = \hat{S}\vec{\varphi} + \hat{F}_p\vec{\varphi} - \hat{R}\vec{\varphi} - \hat{L}\vec{\varphi} + \hat{E}\vec{C}_i \\ (\alpha + \lambda_i)C_i = \hat{F}_{d_i}\vec{\varphi} \quad i = 1, \dots, R, \end{cases} \quad (6)$$

where  $C_i$  is the  $i$ -th precursor concentration,  $\lambda_i$  is the decay constant of the precursor family  $i$ ,  $\hat{F}_d$  is the delayed fission operator and  $\hat{E}$  is the delayed neutron emission density. This second formulation of the  $\alpha$  eigenproblem has deserved recently a lot of attention, regarding both the algorithmic approach to its solution [1] and the characterization of the eigenvalue spectrum [2].

According to the kind of eigenproblem that is formulated and solved, one can retrieve different information on the system. The  $\kappa$ -modes are usually employed as a basis to reconstruct the flux of a perturbed system [7], while the  $\alpha$ -modes are more useful for expanding the flux in time-dependent problems. Other applications are related to the design and interpretation of experiments using the eigenvalues and their separation [8].

In addition to the practical applications, the study of the properties of the different eigenproblem spectra is significant also for theoretical interests: its properties – for instance the existence of complex eigenvalues [9], of a continuous part of the spectrum [10] or the eigenvalues spacing – are influenced by both the physical properties of the system under consideration and by the mathematical scheme employed to solve the problem.

In this work we present the main results about the eigenvalue formulations of the 1D plane geometry transport equation approximated numerically with the  $P_N$  method [11]. In the following sections, an analysis of the convergence pattern of the various eigenvalue formulations with respect to the different types of boundary conditions and to the order employed to truncate the  $P_N$  equations is carried out. Then, the properties of the spectrum of the different eigenproblem formulations are discussed, highlighting also the effects related to the medium heterogeneity and to the adoption of multi-group approximation.

### **3. NUMERICAL SETUP OF THE PROBLEM AND VERIFICATION**

Thanks to the simplified geometry, the different operators appearing in equation (1) can be assembled directly and manipulated in order to retrieve the various eigenvalue problem formulations. To perform these tasks, we exploited the great flexibility offered by the object-oriented programming paradigm implementing a set of classes in Matlab that discretises the transport equation, builds the operators, combines them and looks for a user-defined number of eigenpairs. The code has been verified against other numerical results found in literature for the different eigenvalue formulations in some cases like two-group homogeneous and heterogeneous media. The eigenvalues are in good agreement ( $<50$  pcm for the fundamental one) with the reference ones [12][13][14].

The implementation of the equation numerical solver arises two interesting facts about the  $P_N$  approximation. The first one is related to the fact that the spatial discretisation, performed using the centred finite difference scheme, requires that two staggered grids are defined for the even and odd equations in order to ensure consistency with the continuous equations.

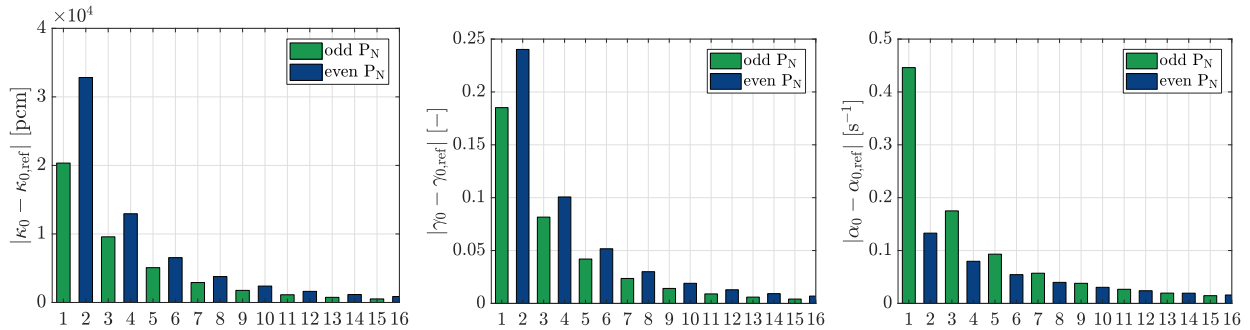
The other observation comes out when dealing with boundary conditions (BCs): while Marshak BCs require that the odd moments vanish for the incoming directions on the two boundaries, Mark BCs require that the incoming angular flux vanishes for a certain set of discrete directions  $\mu_i$ . The choice of these directions is quite natural for odd values of  $N$ . Since the roots of the Legendre polynomial  $L_{N+1}$  are symmetrically distributed in the range  $[-1, 1]$ , the  $\mu_i$  values are usually taken from this set. Moreover, this set of  $\mu_i$  has also the nice feature of providing the equivalence

between  $P_N$  and  $S_{N+1}$ . On the other hand, the choice of  $\mu_i$  in case of even  $N$  leaves a certain freedom, since one can choose the set of  $\mu_i$  as roots of  $L_N$  (i.e. the same of the previous odd order) or  $L_{N+1}$ , removing  $\mu = 0$  as it would not yield a meaningful condition since the particles moving along this directions would not cross the boundaries (i.e., the problem is no longer differential).

The observation on the Mark BCs in case of even  $N$  is related to the fact that, moving from the odd to the subsequent even approximation, the order of the system of equations does not change, as already pointed out in the past by Davison [15]. This feature allows to recast the even  $P_N$  equations into a set of  $P_{N-1}$  equations with modified coefficients. The influence of both the approximation orders and the BCs choice is briefly discussed in the next section.

#### 4. ANGULAR ORDER CONVERGENCE

The convergence behaviour of the eigenvalues  $\kappa$ ,  $\gamma$  and  $\alpha$  can be quite different depending on the boundary conditions imposed. An inspection of figures 1-3 would allow to draw some preliminary conclusions on the  $P_N$  convergence properties. At first, it is interesting to notice that, moving

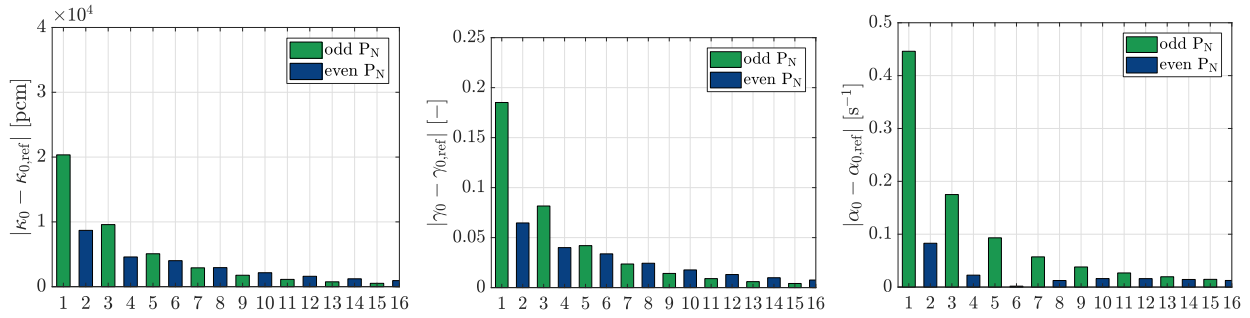


**Figure 1:**  $\kappa$ ,  $\gamma$ , and  $\alpha$  eigenvalues convergence as a function of the angular approximation order with Mark BCs using the roots of  $L_N$ .

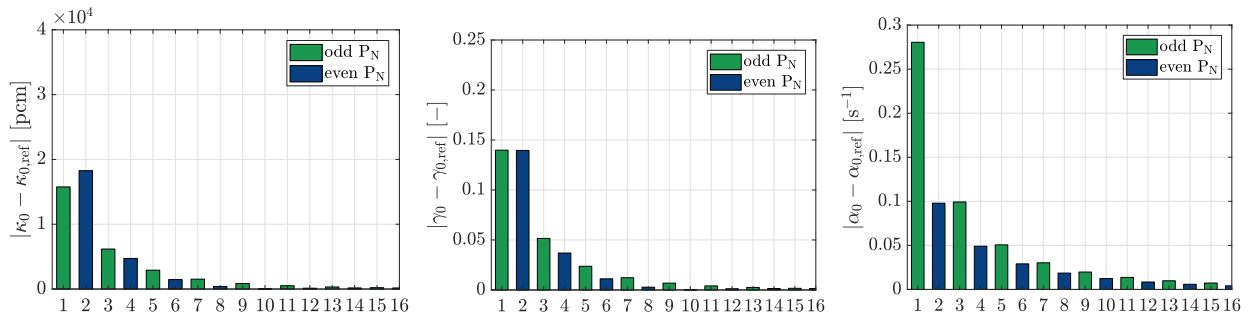
from  $P_1$  to  $P_2$ , the absolute error of  $\kappa$  and  $\gamma$  with respect to the reference solution (evaluated using a  $P_{63}$  for all the three BCs cases) grows for the case of Mark BCs evaluated in the roots of  $L_N$ , decreases for the case of Mark BCs computed in the roots of  $L_{N+1}$  and is about the same with Marshak BCs. As regards  $\alpha$ , the convergence trend from  $P_1$  to  $P_2$  shows that, on the contrary of what is common believed, the even order approximations can provide a significant improvement with respect to the previous odd order. This fact supports the idea that, even if the even  $P_N$  has the same rank of the odd system, the modified coefficients are more representative of the physics described by the problem.

#### 5. EIGENVALUE SPECTRUM

In this section, we use the code to investigate some of the features of the numerical spectrum of different eigenvalue problems presented in section 2, Results for  $\delta$  are not reported, as their behaviour requires a deeper analytical investigation. This fact is in principle not so surprising if one recall the physical meaning of the various eigenvalue. For instance, the  $\kappa$  eigenvalue meaning



**Figure 2:**  $\kappa$ ,  $\gamma$ , and  $\alpha$  eigenvalues convergence as a function of the angular approximation order with Mark BCs using the roots of  $\mathcal{L}_{N+1}$ .



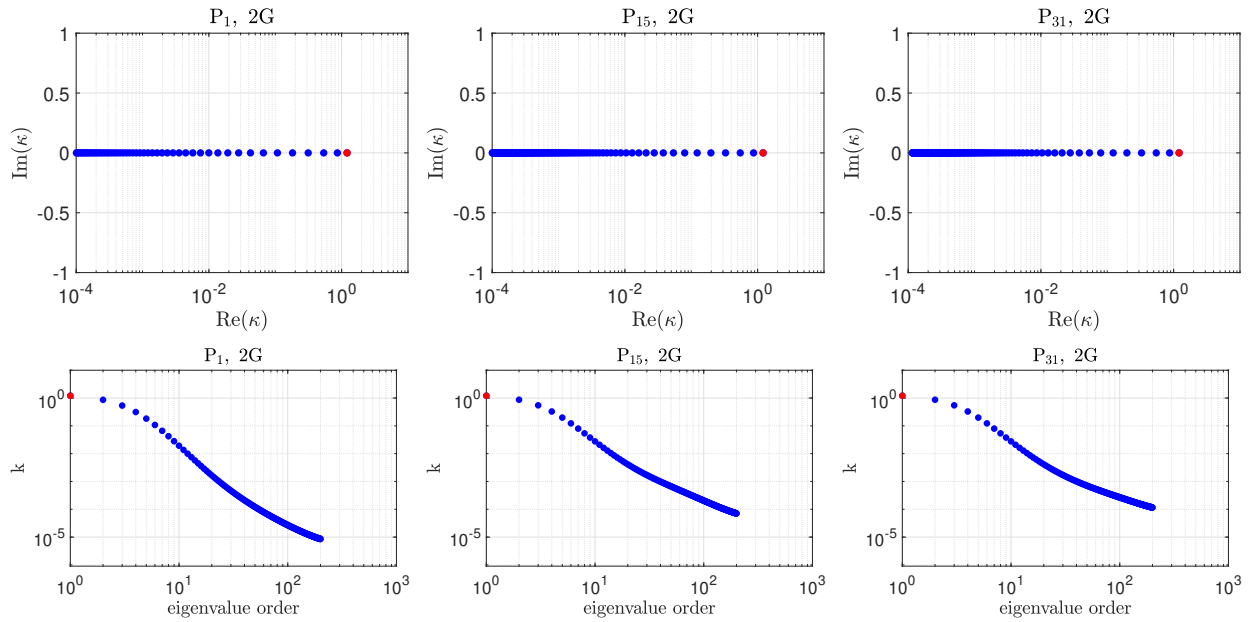
**Figure 3:**  $\kappa$ ,  $\gamma$ , and  $\alpha$  eigenvalues convergence as a function of the angular approximation order with Marshak BCs.

is the inverse of the fractional variation of the fissile density which is required to make the system critical. Then, a real  $\kappa$  value smaller than 1 is expected for a subcritical system and viceversa for a supercritical one. As far as  $\delta$  eigenvalue is concerned - as it is related to the number of collisions - we simply remark its physical meaning is the inverse of the fractional variation of the global density of the system which is needed to make it critical: it is not obvious that a real positive value exists as a solution for this rather unusual problem.

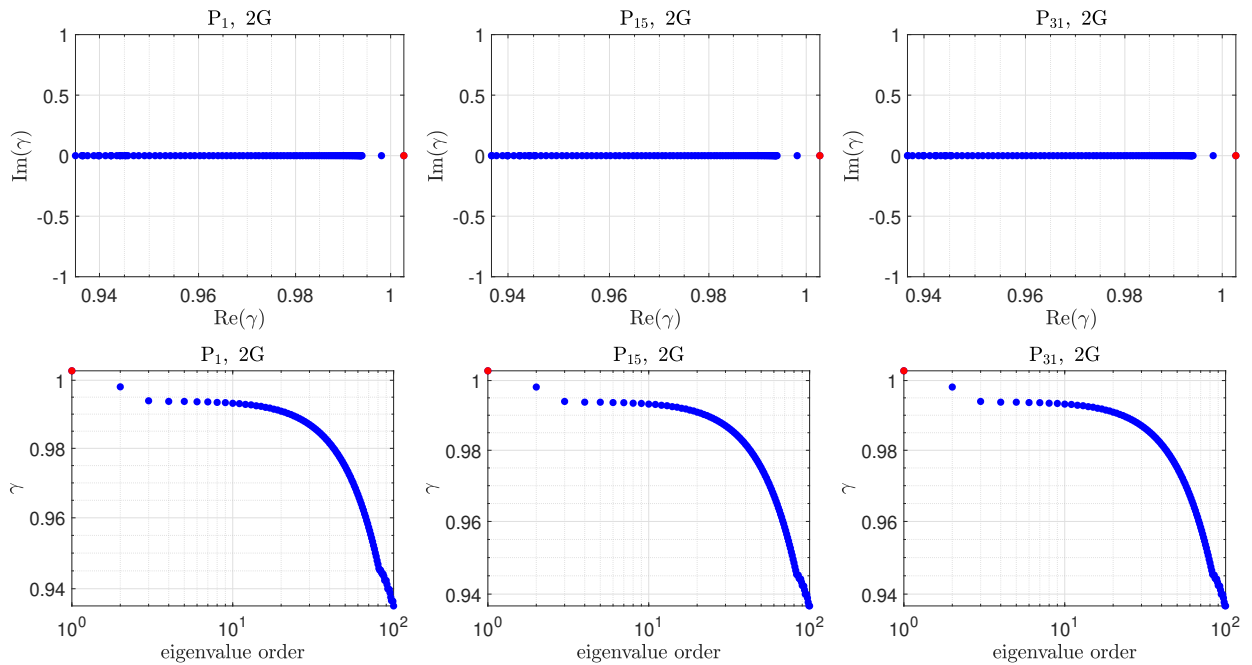
In Figures 4-7 the spectra evaluated are reported, with a red dot indicating the fundamental eigenvalue.

Figure 4 shows the behaviour of the  $\kappa$  spectrum for increasing values of the angular approximation order in case of a symmetrically reflected system, with a reflector thickness of 100 cm of  $H_2O$  and 20 cm of fissile material (for the two-group data, please refer to the URRb-H<sub>2</sub>Oa(1)-2-0-SL benchmark in [14]). Increasing the  $P_N$  order does not seem to change the behaviour of the spectrum, which attempts to reproduce the continuous spectrum for reflected systems

The  $\alpha$  delayed spectrum is evaluated for a homogeneous slab with Mark boundary conditions (for the two-group data employed, please refer to the PU-2-0-SL benchmark in [14]).

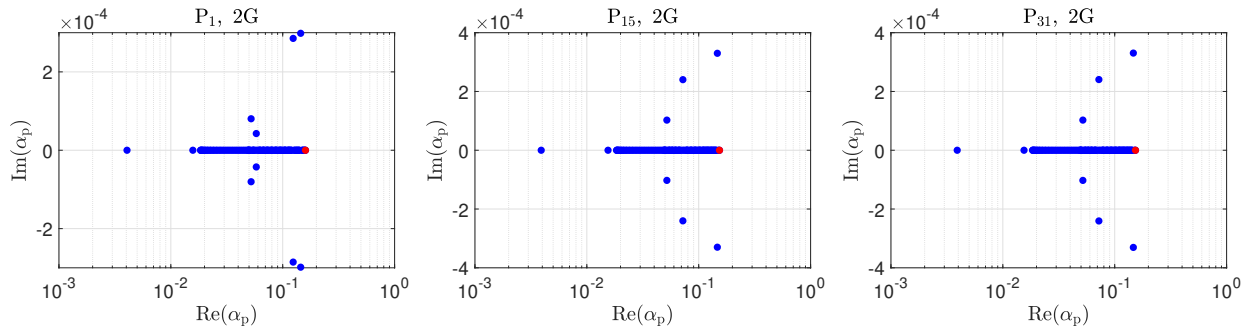


**Figure 4:**  $\kappa$  eigenvalue distribution for a symmetrically re ected system.



**Figure 5:**  $\gamma$  eigenvalue distribution for a symmetrically re ected system.

The comparison of the results highlights that the  $\gamma$  spectrum has real eigenvalues, as for  $\kappa$ , while the time-eigenvalue spectrum shows, as expected, a distribution in the complex plane with clustering

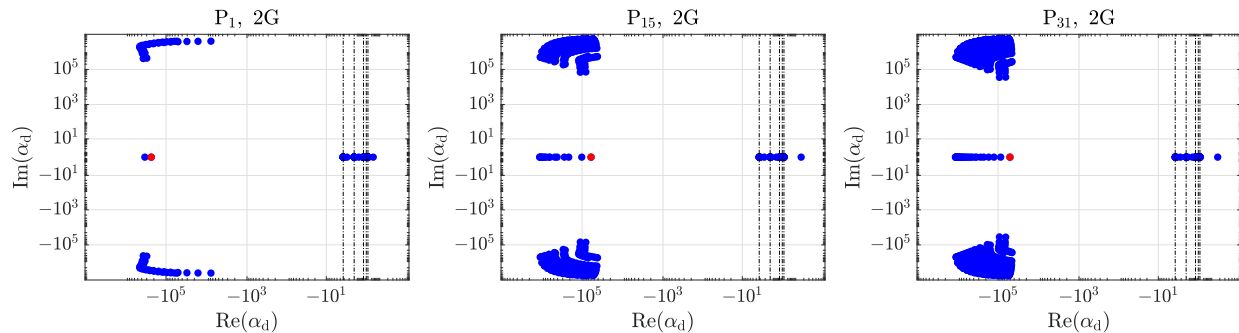


**Figure 6:**  $\alpha$  prompt eigenvalue distribution for a symmetrically reflected system.

of eigenvalues as observed in other situation [2].

## 6. CONCLUDING REMARKS

In this work, the eigenvalue problem associated to the solution of the neutron balance equation is analyzed, focusing on the different formulations that can be given to the problem, with their physical significance. The role of the method adopted for its solution is discussed, focusing on the  $P_N$  method and pointing out at some peculiar effects regarding the angular expansion order which deserves further investigations to assess the suitability of even-order PN for the solution of reactor physics-relevant problems.



**Figure 7:**  $\alpha$  eigenvalue distribution for a homogeneous system including delayed neutrons. The dotted lines are the values of  $\lambda_i$ .

## 7. ACKNOWLEDGMENTS

The Authors are grateful to the reviewer for their very insightful comments and useful suggestions.



## REFERENCES

- [1] Andrea Zoia, Emeric Brun, and Fausto Malvagi. Alpha eigenvalue calculations with TRIPOLI-4®. *Annals of Nuclear Energy*, 63:276 – 284, 2014.
- [2] Richard Sanchez, Daniele Tomatis, Igor Zmijarevic, and Han Gyu Joo. Analysis of alpha modes in multigroup diffusion. *Nuclear Engineering and Technology*, 1259-1268 – 49, 2017.
- [3] G.B. Bruna and K. Burn. Impact of the heavy steel reflector of a current large PWR on some safety features. In *Proceedings of the TOPSAFE 2012 Conference*, Helsinki, Finland, April 22-26 2012.
- [4] Nicolò Abrate, Giovanni Bruna, Sandra Dulla, and Piero Ravetto. Assessment of numerical methods for the evaluation of higher-order harmonics in diffusion theory. *Annals of Nuclear Energy*, 128:455 – 470, 2019.
- [5] N. Abrate, M. Burrone, S. Dulla, P. Ravetto, and P. Saracco. Eigenvalue formulations for the spherical harmonics approximation to the neutron transport equation. In *Proceedings of the ICTT 2019 Conference*, Paris, France, September 23-27 2019.
- [6] D. G. Cacuci, Y. Ronen, Z. Shayer, J. J. Wagschal, and Y. Yeivin. Eigenvalue-Dependent Neutron Energy Spectra: Definitions, Analyses, and Applications. *Nuclear Science and Engineering*, 81(3):432–442, 2017.
- [7] Augusto Gandini. On the Standard Perturbation Theory. *Nuclear Science and Engineering*, 79(4):426–430, dec 1981.
- [8] N. Chentre, P. Saracco, S. Dulla, and P. Ravetto. On the prompt time eigenvalue estimation for subcritical multiplying systems. *Annals of Nuclear Energy*, 132:172–180, 2019.
- [9] Paolo Saracco, Sandra Dulla, and Piero Ravetto. On the spectrum of the multigroup diffusion equations. *Progress in Nuclear Energy*, 59:86–95, 2012.
- [10] S. Dulla, P. Ravetto, and P. Saracco. The time eigenvalue spectrum for nuclear reactors in multi-group diffusion theory. *European Physical Journal Plus*, 133(9):1–24, 2018.
- [11] R. V. Meghreblian and D. K. Holmes. *Reactor analysis*. 1963.
- [12] R. S. Modak, D. C. Sahni, and S. D. Paranjape. Evaluation of higher k-eigenvalues of the neutron transport equation by SN method. *Annals of Nuclear Energy*, 22: 359–366 1995.
- [13] D. Lathouwers. Iterative computation of time-eigenvalues of the neutron transport equation. *Annals of Nuclear Energy*, 30(17):1793–1806, 2003.
- [14] A. Sood, A. R. Forster, and K. D. Parsons. Analytical benchmark test set for criticality code verification. *Progress in Nuclear Energy*, 42(1):55–106, 2003.
- [15] B. Davison. *Neutron transport theory*. International series of monographs on physics. Clarendon Press, 1958.