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STUDY OF THE EIGENVALUE SPECTRA OF THE NEUTRON TRANSPORT PROBLEM IN PN APPROXIMATION / Saracco, Paolo; Abrate, Nicolo; Burrone, Miriam; Dulla, Sandra; Ravetto, Piero. - ELETTRONICO. - (2020). (International Conference PHYSOR 2020 Cambridge).

Availability:

This version is available at: 11583/2841543 since: 2020-10-16T11:17:58Z

Publisher:

American Nuclear Society

Published

DOI:

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STUDY OF THE EIGENVALUE SPECTRA OF THE NEUTRON TRANSPORT PROBLEM IN P_N APPROXIMATION

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ABSTRACT

The study of the steady-state solutions of neutron transport equation requires the introduction of appropriate eigenvalues: this can be done in various different ways by changing each of the operators in the transport equation; such modifications can be physically viewed as a variation of the corresponding macroscopic cross sections only, so making the different (generalized) eigenvalue problems non-equivalent. In this paper the eigenvalue problem associated to the time-dependent problem (α eigenvalue), also in the presence of delayed emissions is evaluated. The properties of associated spectra can give different insight into the physics of the problem.

KEYWORDS: P_N approximation, eigenvalue spectra, time eigenvalue, collision eigenvalue, effective multiplication parameter

1. INTRODUCTION

In reactor physics, the study of the the neutronics of multiplying systems requires the solution of the transport equation in steady-state condition. In particular, the determination of the criticality condition is associated to the solution of an eigenvalue formulation of the neutron balance problem, thus allowing a non-trivial solution.

In general terms, such eigenvalue problem can be formulated in various alternative forms, each allowing different physical interpretations. The standard criticality problem is customarily approached by introducing a *multiplication eigenvalue* in correspondence of the fission term. In more recent times the α *eigenvalue* problem has gained larger interest [1,2], thanks to the associated significance in terms of asymptotic evolution of the system.

It must be pointed out as well that the solution of the eigenvalue problem for the neutron balance model can provide, in general, a set of eigenvalues associated to the corresponding eigenfunctions, which can constitute a useful basis for projection techniques of relevance in reactor physics analyses, such as higher-order perturbation theory [3,4]. It is therefore extremely interesting to approach the analysis of the eigenvalue formulation of the neutron transport model, focusing on both the possible different formulations which can be given to the problem and the methods adopted for its solution.

In this work, different formulations of the eigenvalue problems are compared in terms of characteristics of the resulting model and physical significance. The numerical solution is then achieved adopting a P_N approximation for the angular variable, focusing on some specific features of the solution when comparing odd- and even-order expansions [5]. The solution approach adopted, applied to simplified configurations, allows to calculate a set of eigenvalues, thus accessing the information related to higher-order harmonics. The eigenvalue spectra for the different formulations of the problem will be compared and discussed.

2. Eigenvalue problem formulations

According to what is the physical phenomenon of interest for a certain system, the eigenvalue problem associated to the transport (or diffusion) equation,

$$\hat{T}\vec{\varphi} + \hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} = \hat{S}\vec{\varphi} + \hat{F}\vec{\varphi}, \quad (1)$$

where \hat{T} is the time operator, \hat{L} is the leakage, \hat{R} is the removal, \hat{S} is the scattering, \hat{F} is the total fission and $\vec{\varphi}$ is the flux, can be formulated in different ways, each one providing a different interpretation of the problem [6].

The most popular approach to find the steady state neutron distribution is the one traditionally introduced by Fermi, i.e. the criticality eigenproblem, where the κ eigenvalue is introduced in the steady state balance equation as a "tuning" parameter for the fission operator,

$$\hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} - \hat{S}\vec{\varphi} = \frac{1}{\kappa}\hat{F}\vec{\varphi}. \quad (2)$$

A similar approach for the stationary equation focuses on the collisions, dividing both the scattering and the fission operators by the γ eigenvalue,

$$\hat{L}\vec{\varphi} + \hat{R}\vec{\varphi} = \frac{1}{\gamma}(\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi}). \quad (3)$$

A more exotic and less explored alternative to study the steady state chain reaction is to look for the material density allowing the system to become critical: this objective is achieved through the introduction of a δ eigenvalue for all collision terms in the balance equation:

$$\hat{L}\vec{\varphi} = \frac{1}{\delta}(\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi} - \hat{R}\vec{\varphi}). \quad (4)$$

As far as the time-dependent problem is concerned, assuming suitable initial conditions, it is possible to substitute the time operator \hat{T} with the so-called *time eigenvalue*, α , which has the physical meaning of the inverse of the reactor period:

$$\hat{S}\vec{\varphi} + \hat{F}\vec{\varphi} - \hat{R}\vec{\varphi} - \hat{L}\vec{\varphi} = \frac{\alpha}{v}\vec{\varphi}. \quad (5)$$

Such formulation, which can also be interpreted as a Laplace transform of the time-dependent balance model, is more correctly representing the asymptotic behavior of a nuclear system when delayed fission emissions are included, leading to:

$$\begin{cases} \frac{\alpha}{v}\vec{\varphi} = \hat{S}\vec{\varphi} + \hat{F}_p\vec{\varphi} - \hat{R}\vec{\varphi} - \hat{L}\vec{\varphi} + \hat{E}\vec{C}_i \\ (\alpha + \lambda_i)C_i = \hat{F}_{d_i}\vec{\varphi} \quad i = 1, \dots, R, \end{cases} \quad (6)$$

where C_i is the i -th precursor concentration, λ_i is the decay constant of the precursor family i , \hat{F}_d is the delayed fission operator and \hat{E} is the delayed neutron emission density. This second formulation of the α eigenproblem has deserved recently a lot of attention, regarding both the algorithmic approach to its solution [1] and the characterization of the eigenvalue spectrum [2].

According to the kind of eigenproblem that is formulated and solved, one can retrieve different information on the system. The κ -modes are usually employed as a basis to reconstruct the flux of a perturbed system [7], while the α -modes are more useful for expanding the flux in time-dependent problems. Other applications are related to the design and interpretation of experiments using the eigenvalues and their separation [8].

In addition to the practical applications, the study of the properties of the different eigenproblem spectra is significant also for theoretical interests: its properties – for instance the existence of complex eigenvalues [9], of a continuous part of the spectrum [10] or the eigenvalues spacing – are influenced by both the physical properties of the system under consideration and by the mathematical scheme employed to solve the problem.

In this work we present the main results about the eigenvalue formulations of the 1D plane geometry transport equation approximated numerically with the P_N method [11]. In the following sections, an analysis of the convergence pattern of the various eigenvalue formulations with respect to the different types of boundary conditions and to the order employed to truncate the P_N equations is carried out. Then, the properties of the spectrum of the different eigenproblem formulations are discussed, highlighting also the effects related to the medium heterogeneity and to the adoption of multi-group approximation.

3. NUMERICAL SETUP OF THE PROBLEM AND VERIFICATION

Thanks to the simplified geometry, the different operators appearing in equation (1) can be assembled directly and manipulated in order to retrieve the various eigenvalue problem formulations. To perform these tasks, we exploited the great flexibility offered by the object-oriented programming paradigm implementing a set of classes in Matlab that discretises the transport equation, builds the operators, combines them and looks for a user-defined number of eigenpairs. The code has been verified against other numerical results found in literature for the different eigenvalue formulations in some cases like two-group homogeneous and heterogeneous media. The eigenvalues are in good agreement (<50 pcm for the fundamental one) with the reference ones [12][13][14].

The implementation of the equation numerical solver arises two interesting facts about the P_N approximation. The first one is related to the fact that the spatial discretisation, performed using the centred finite difference scheme, requires that two staggered grids are defined for the even and odd equations in order to ensure consistency with the continuous equations.

The other observation comes out when dealing with boundary conditions (BCs): while Marshak BCs require that the odd moments vanish for the incoming directions on the two boundaries, Mark BCs require that the incoming angular flux vanishes for a certain set of discrete directions μ_i . The choice of these directions is quite natural for odd values of N . Since the roots of the Legendre polynomial L_{N+1} are symmetrically distributed in the range $[-1, 1]$, the μ_i values are usually taken from this set. Moreover, this set of μ_i has also the nice feature of providing the equivalence

between P_N and S_{N+1} . On the other hand, the choice of μ_i in case of even N leaves a certain freedom, since one can choose the set of μ_i as roots of L_N (i.e. the same of the previous odd order) or L_{N+1} , removing $\mu = 0$ as it would not yield a meaningful condition since the particles moving along this directions would not cross the boundaries (i.e., the problem is no longer differential).

The observation on the Mark BCs in case of even N is related to the fact that, moving from the odd to the subsequent even approximation, the order of the system of equations does not change, as already pointed out in the past by Davison [15]. This feature allows to recast the even P_N equations into a set of P_{N-1} equations with modified coefficients. The influence of both the approximation orders and the BCs choice is briefly discussed in the next section.

4. ANGULAR ORDER CONVERGENCE

The convergence behaviour of the eigenvalues κ , γ and α can be quite different depending on the boundary conditions imposed. An inspection of figures 1-3 would allow to draw some preliminary conclusions on the P_N convergence properties. At first, it is interesting to notice that, moving

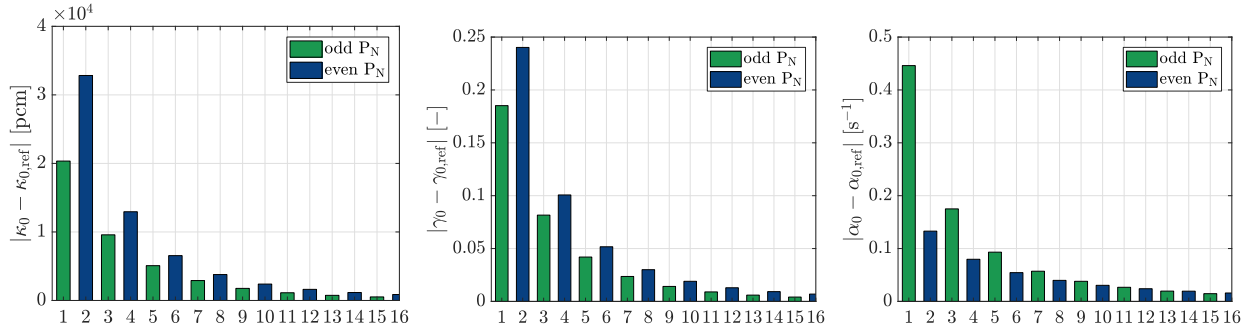


Figure 1: κ , γ , and α eigenvalues convergence as a function of the angular approximation order with Mark BCs using the roots of L_N .

from P_1 to P_2 , the absolute error of κ and γ with respect to the reference solution (evaluated using a P_{63} for all the three BCs cases) grows for the case of Mark BCs evaluated in the roots of L_N , decreases for the case of Mark BCs computed in the roots of L_{N+1} and is about the same with Marshak BCs. As regards α , the convergence trend from P_1 to P_2 shows that, on the contrary of what is common believed, the even order approximations can provide a significant improvement with respect to the previous odd order. This fact supports the idea that, even if the even P_N has the same rank of the odd system, the modified coefficients are more representative of the physics described by the problem.

5. EIGENVALUE SPECTRUM

In this section, we use the code to investigate some of the features of the numerical spectrum of different eigenvalue problems presented in section 2, Results for δ are not reported, as their behaviour requires a deeper analytical investigation. This fact is in principle not so surprising if one recall the physical meaning of the various eigenvalue. For instance, the κ eigenvalue meaning

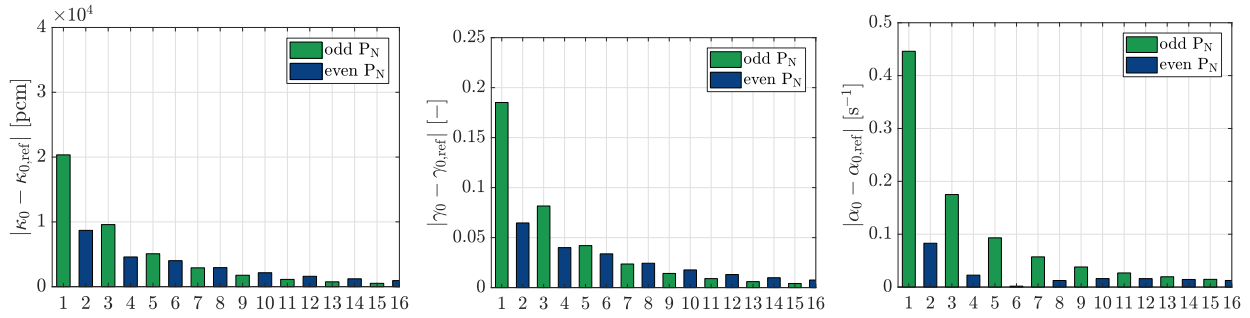


Figure 2: κ , γ , and α eigenvalues convergence as a function of the angular approximation order with Mark BCs using the roots of L_{N+1} .

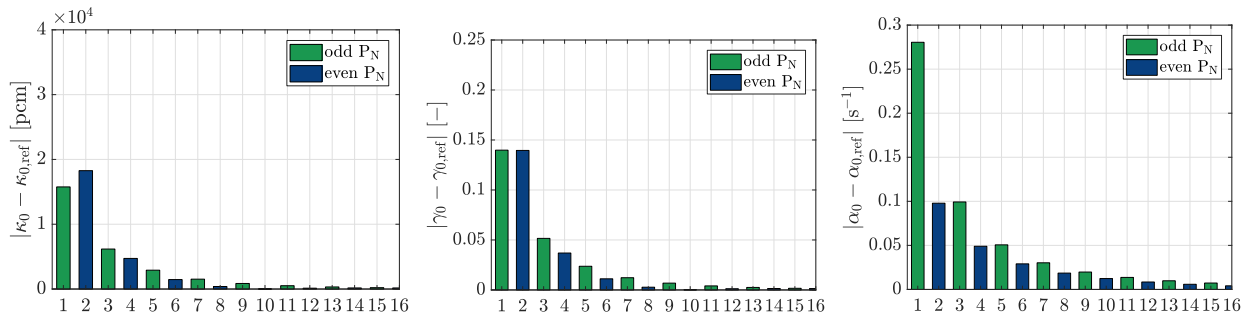


Figure 3: κ , γ , and α eigenvalues convergence as a function of the angular approximation order with Marshak BCs.

is the inverse of the fractional variation of the fissile density which is required to make the system critical. Then, a real κ value smaller than 1 is expected for a subcritical system and viceversa for a supercritical one. As far as δ eigenvalue is concerned - as it is related to the number of collisions - we simply remark its physical meaning is the inverse of the fractional variation of the global density of the system which is needed to make it critical: it is not obvious that a real positive value exists as a solution for this rather unusual problem.

In Figures 4-7 the spectra evaluated are reported, with a red dot indicating the fundamental eigenvalue.

Figure 4 shows the behaviour of the κ spectrum for increasing values of the angular approximation order in case of a symmetrically reflected system, with a reflector thickness of 100 cm of H_2O and 20 cm of fissile material (for the two-group data, please refer to the URRb-H₂Oa(1)-2-0-SL benchmark in [14]). Increasing the P_N order does not seem to change the behaviour of the spectrum, which attempts to reproduce the continuous spectrum for reflected systems

The α delayed spectrum is evaluated for a homogeneous slab with Mark boundary conditions (for the two-group data employed, please refer to the PU-2-0-SL benchmark in [14]).

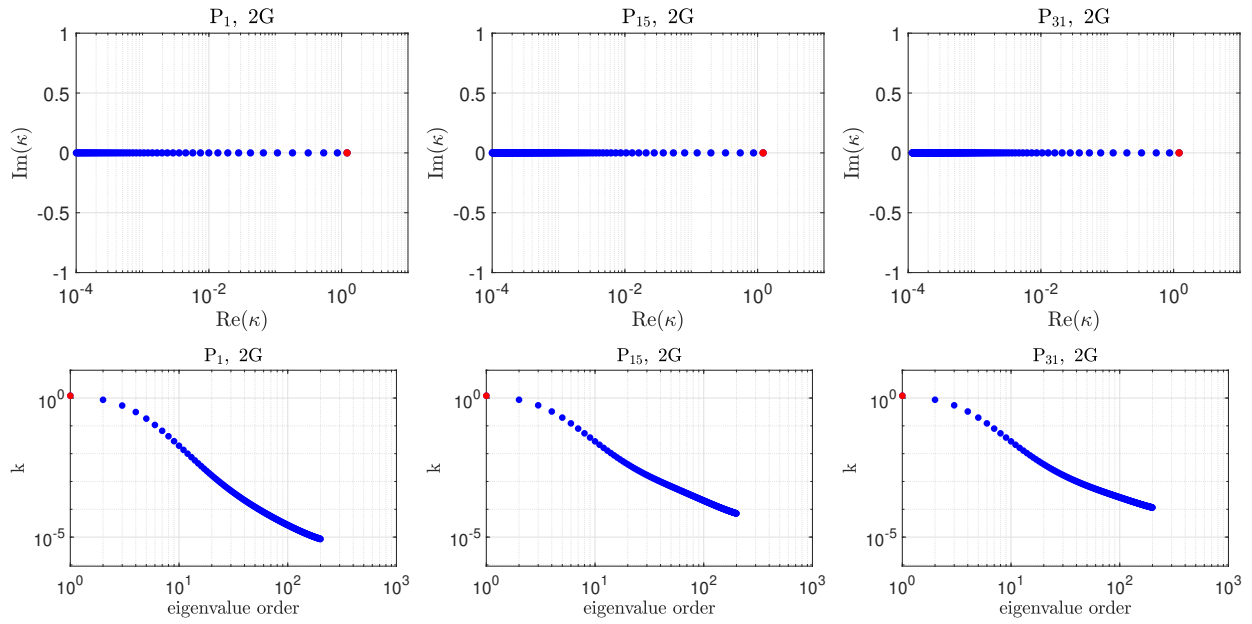


Figure 4: κ eigenvalue distribution for a symmetrically reflected system.

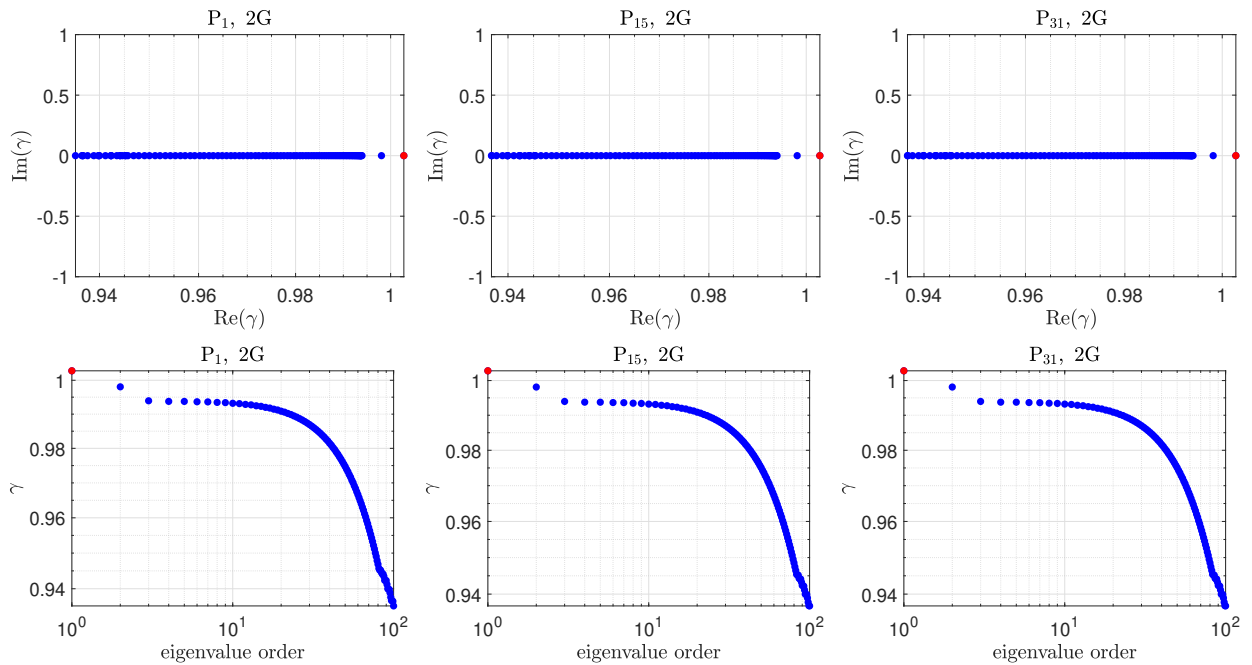


Figure 5: γ eigenvalue distribution for a symmetrically reflected system.

The comparison of the results highlights that the γ spectrum has real eigenvalues, as for κ , while the time-eigenvalue spectrum shows, as expected, a distribution in the complex plane with clustering

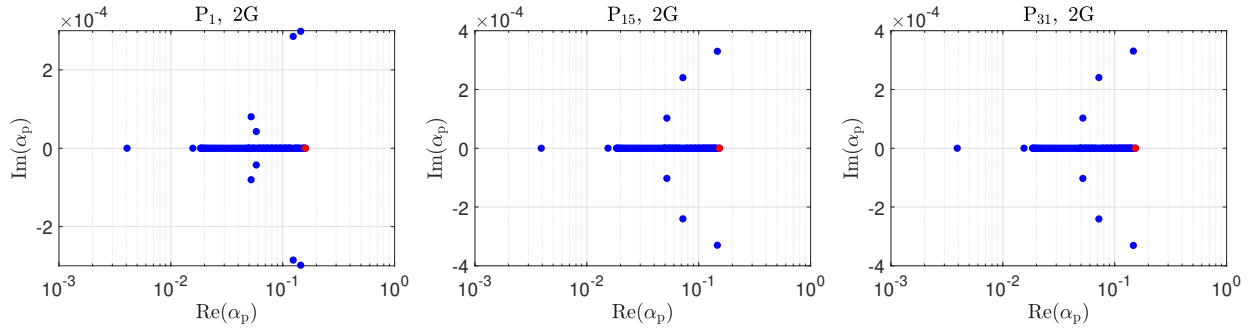


Figure 6: α prompt eigenvalue distribution for a symmetrically reflected system.

of eigenvalues as observed in other situation [2].

6. CONCLUDING REMARKS

In this work, the eigenvalue problem associated to the solution of the neutron balance equation is analyzed, focusing on the different formulations that can be given to the problem, with their physical significance. The role of the method adopted for its solution is discussed, focusing on the P_N method and pointing out at some peculiar effects regarding the angular expansion order which deserves further investigations to assess the suitability of even-order PN for the solution of reactor physics-relevant problems.

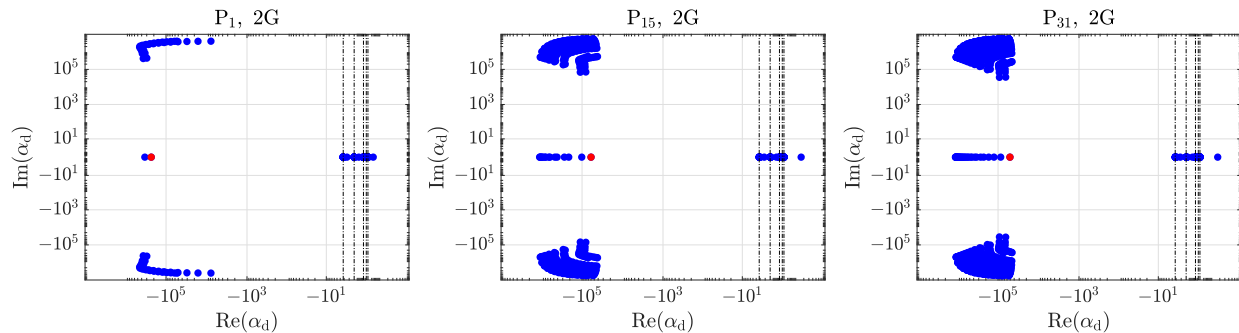


Figure 7: α eigenvalue distribution for a homogeneous system including delayed neutrons. The dotted lines are the values of $-\lambda_i$.

7. ACKNOWLEDGMENTS

The Authors are grateful to the reviewer for their very insightful comments and useful suggestions.

REFERENCES

- [1] Andrea Zoia, Emeric Brun, and Fausto Malvagi. Alpha eigenvalue calculations with TRIPOLI-4®. *Annals of Nuclear Energy*, 63:276 – 284, 2014.
- [2] Richard Sanchez, Daniele Tomatis, Igor Zmijarevic, and Han Gyu Joo. Analysis of alpha modes in multigroup diffusion. *Nuclear Engineering and Technology*, 1259-1268 – 49, 2017.
- [3] G.B. Bruna and K. Burn. Impact of the heavy steel reflector of a current large PWR on some safety features. In *Proceedings of the TOPSAFE 2012 Conference*, Helsinki, Finland, April 22-26 2012.
- [4] Nicolò Abrate, Giovanni Bruna, Sandra Dulla, and Piero Ravetto. Assessment of numerical methods for the evaluation of higher-order harmonics in diffusion theory. *Annals of Nuclear Energy*, 128:455 – 470, 2019.
- [5] N. Abrate, M. Burrone, S. Dulla, P. Ravetto, and P. Saracco. Eigenvalue formulations for the spherical harmonics approximation to the neutron transport equation. In *Proceedings of the ICTT 2019 Conference*, Paris, France, September 23-27 2019.
- [6] D. G. Cacuci, Y. Ronen, Z. Shayer, J. J. Wagschal, and Y. Yeivin. Eigenvalue-Dependent Neutron Energy Spectra: Definitions, Analyses, and Applications. *Nuclear Science and Engineering*, 81(3):432–442, 2017.
- [7] Augusto Gandini. On the Standard Perturbation Theory. *Nuclear Science and Engineering*, 79(4):426–430, dec 1981.
- [8] N. Chentre, P. Saracco, S. Dulla, and P. Ravetto. On the prompt time eigenvalue estimation for subcritical multiplying systems. *Annals of Nuclear Energy*, 132:172–180, 2019.
- [9] Paolo Saracco, Sandra Dulla, and Piero Ravetto. On the spectrum of the multigroup diffusion equations. *Progress in Nuclear Energy*, 59:86–95, 2012.
- [10] S. Dulla, P. Ravetto, and P. Saracco. The time eigenvalue spectrum for nuclear reactors in multi-group diffusion theory. *European Physical Journal Plus*, 133(9):1–24, 2018.
- [11] R. V. Meghreblian and D. K. Holmes. *Reactor analysis*. 1963.
- [12] R. S. Modak, D. C. Sahni, and S. D. Paranjape. Evaluation of higher k-eigenvalues of the neutron transport equation by SN method. *Annals of Nuclear Energy*, 22: 359–366 1995.
- [13] D. Lathouwers. Iterative computation of time-eigenvalues of the neutron transport equation. *Annals of Nuclear Energy*, 30(17):1793–1806, 2003.
- [14] A. Sood, A. R. Forster, and K. D. Parsons. Analytical benchmark test set for criticality code verification. *Progress in Nuclear Energy*, 42(1):55–106, 2003.
- [15] B. Davison. *Neutron transport theory*. International series of monographs on physics. Clarendon Press, 1958.