

Progress in the thermal shield design for the Divertor Tokamak Test (DTT) facility



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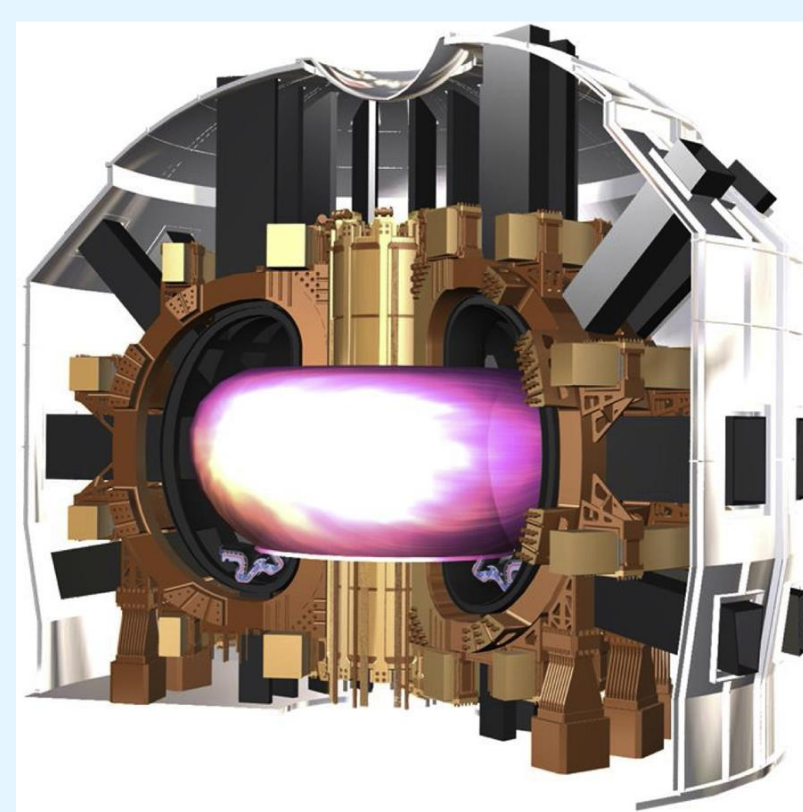
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Introduction

- Divertor Tokamak Test (DTT) under construction at ENEA Frascati research centre, Italy
- High field superconducting (SC) tokamak (6T) carrying 5.5 MA plasma current (~ 100 s plasma pulses) with major radius $R_0 = 2,19$ m
- Thermal shield (THS) needed to minimize the thermal loads on the magnets from the warm and hot components of the tokamak



[Interim Design Report, April 2019]

AIM OF THE PROJECT: investigate alternative power exhaust solutions in EU DEMO relevant conditions

Aim of the work

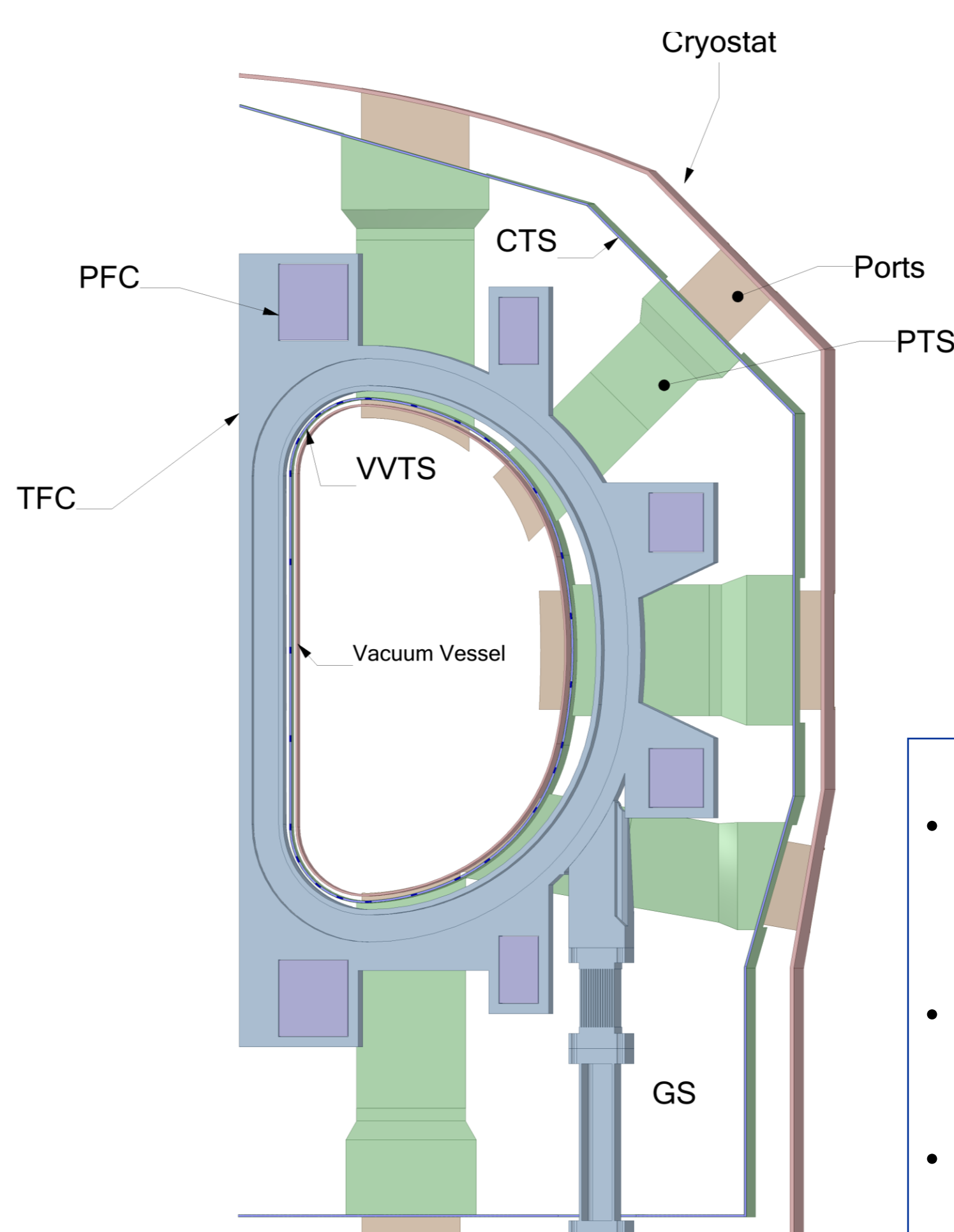
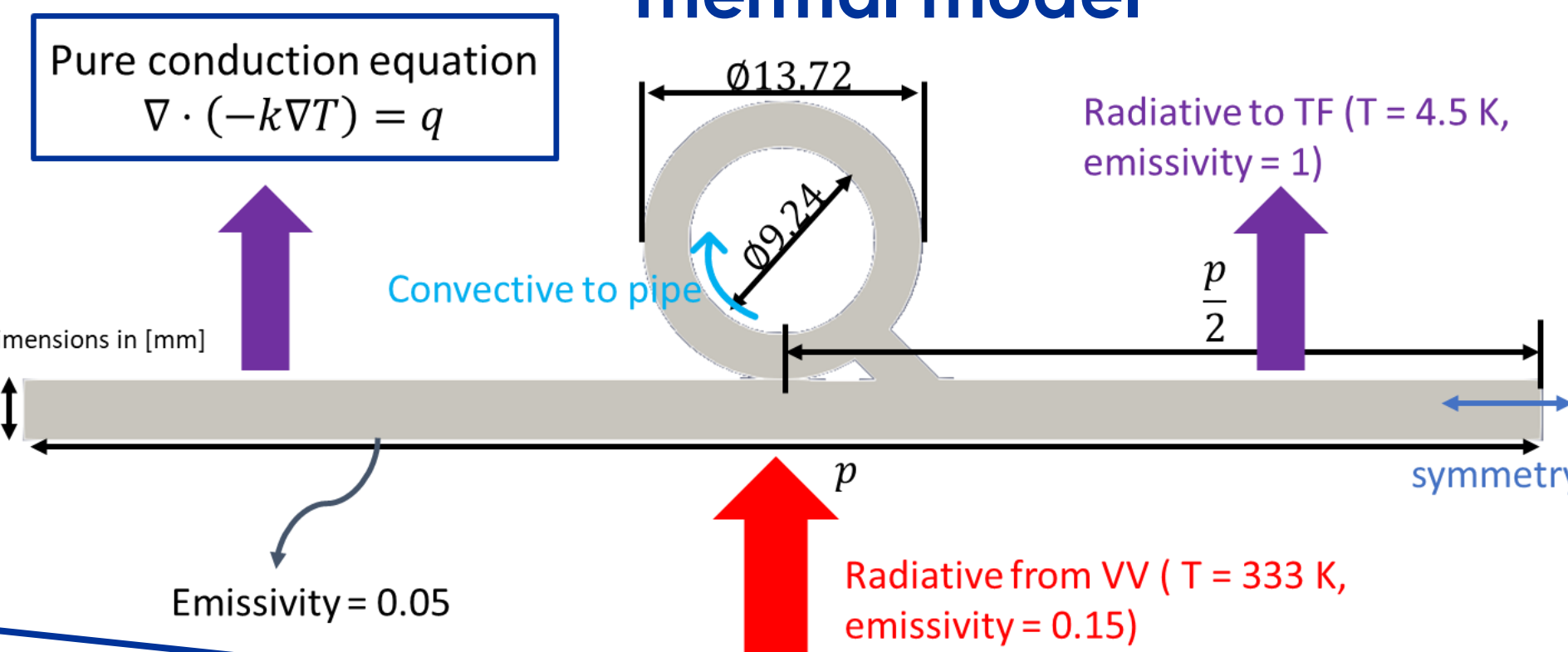
Present the current status of the design of the vacuum vessel THS and the conceptual design of the port and cryostat THS.

Solutions adopted for:

- minimization of the radiative heat transfer
- proper cooling
- mechanical support

Thermal shield (THS) design

- Set of **3 mm thick silver-plated stainless-steel panels** surrounding vacuum vessel (VV), port extensions and cryostat
- Actively cooled by gaseous He at ~20 bar and ~80 K in **pipes laying on the surface of the panels**
- Withstand mechanical (weight, seismic, thermal-contraction) and electro-magnetic loads → **gravity supports**
- Radiative **heat load from the hot components** + conductive heat load due to gravity supports



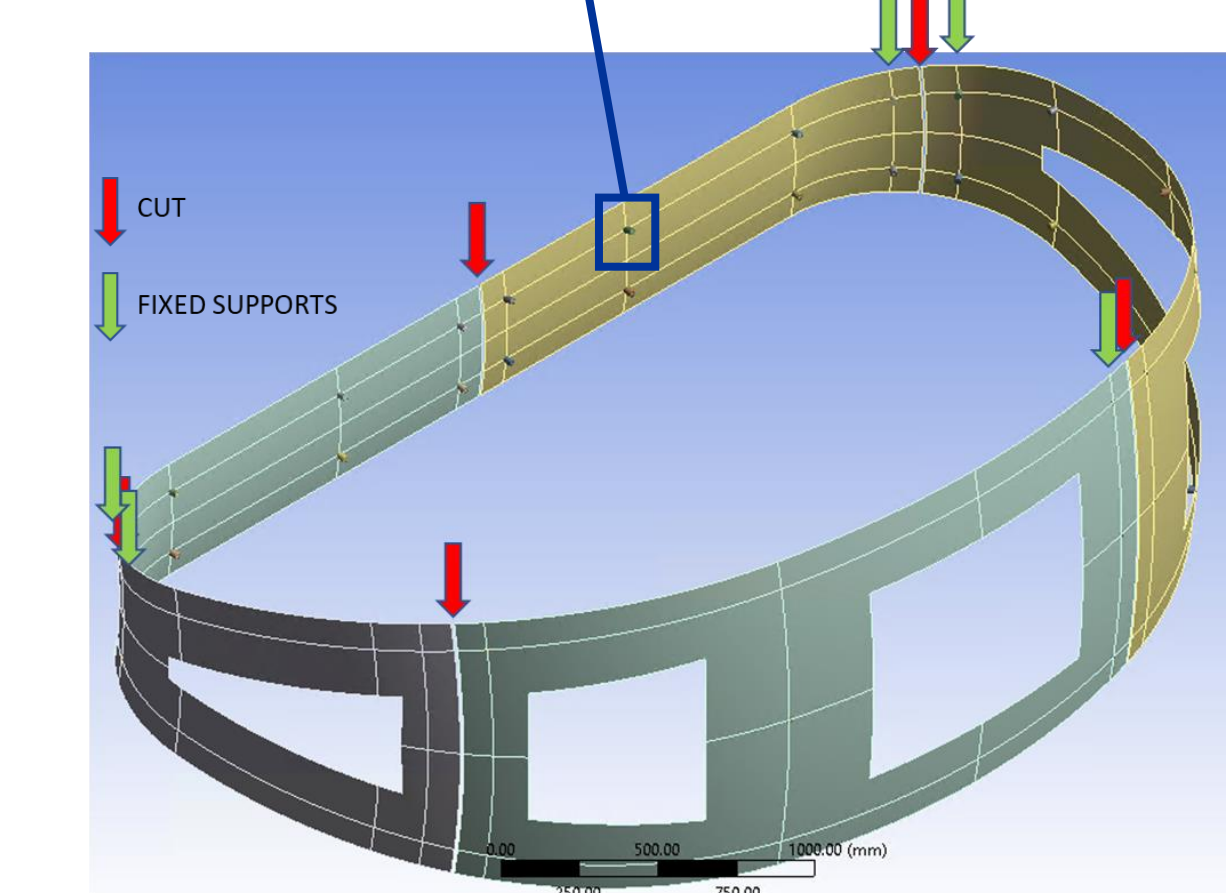
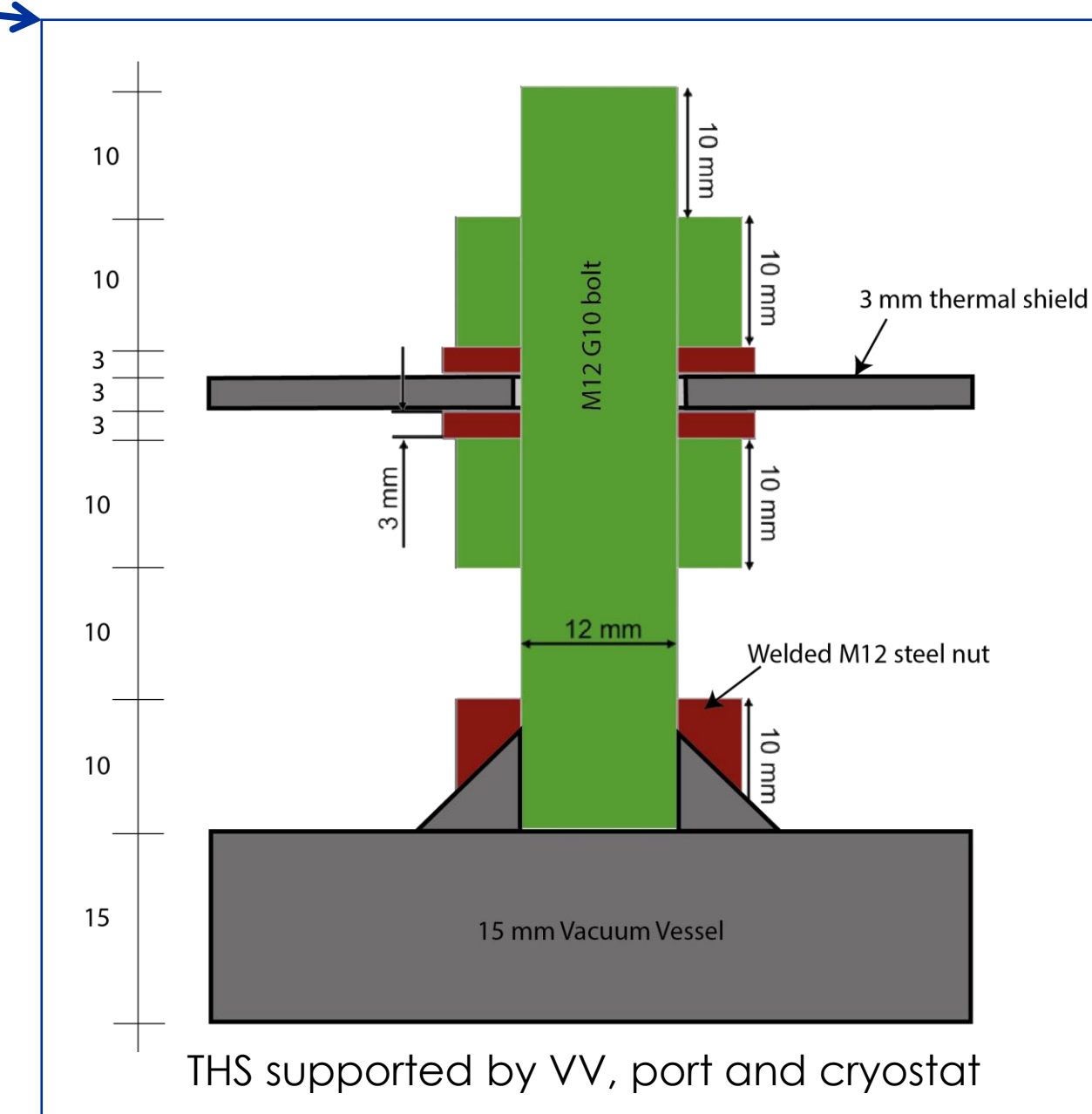
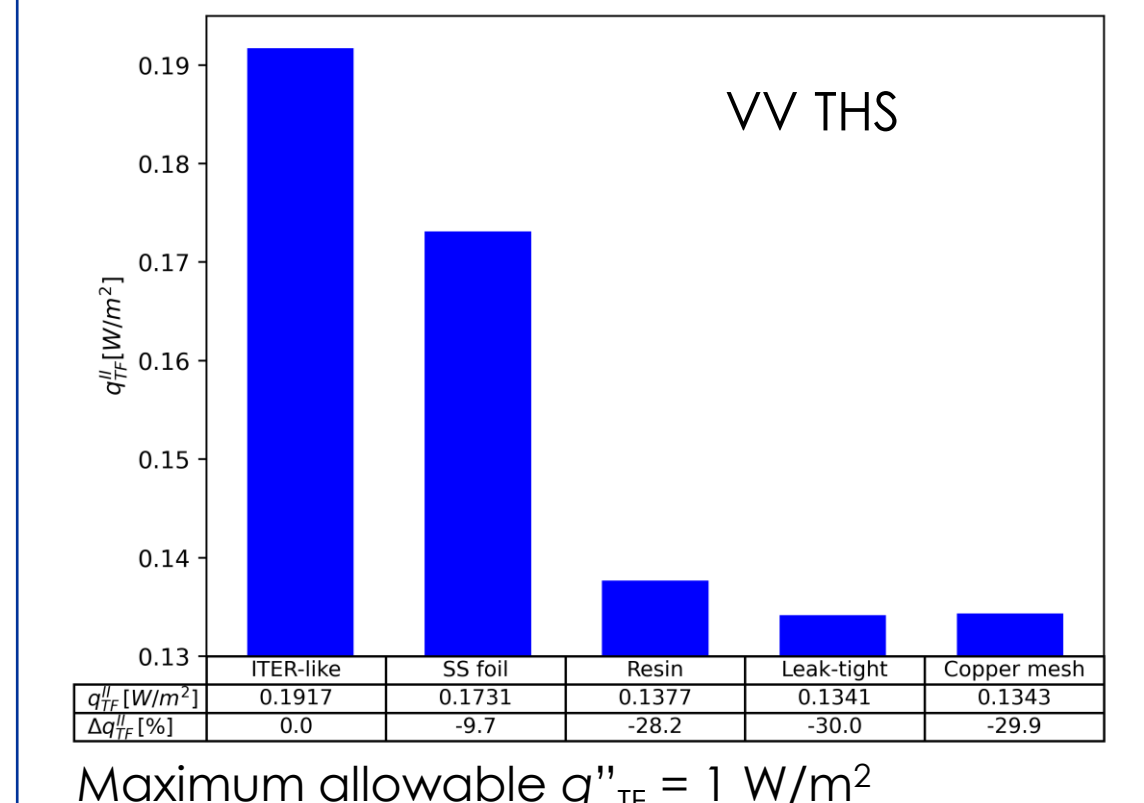
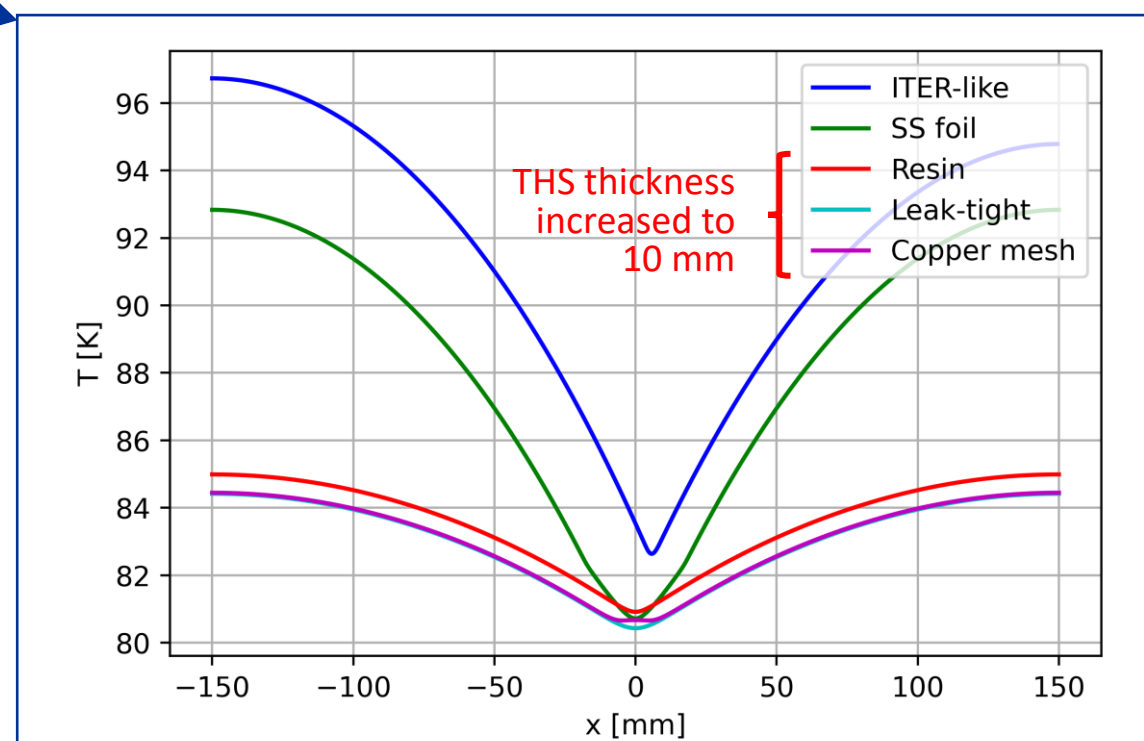
- 340° of VV without port extensions
- VV THS
- Magnets
- Last 20° of VV
- Port extensions with THS already installed
- Cryostat with THS already installed

- ITER-like alternate welding (thin panels to reduce stresses on tubes, direct welding of the pipe wall)
- SS foil: vacuum compatibility and contact reliability to be demonstrated
- High thermal conductivity resin (0.15 W/m/K at 4 K): vacuum compatibility and contact reliability to be demonstrated
- Leak-tight welding: weak vacuum-coolant boundary
- Copper mesh: expensive due to machining of panels, easy shaping of different parts, assembly operations

Option: increase THS thickness only close to the pipe

From ITER TF casing
From ITER TFMC casing
From ITER CSI

[Martovetsky, Naka 2015]



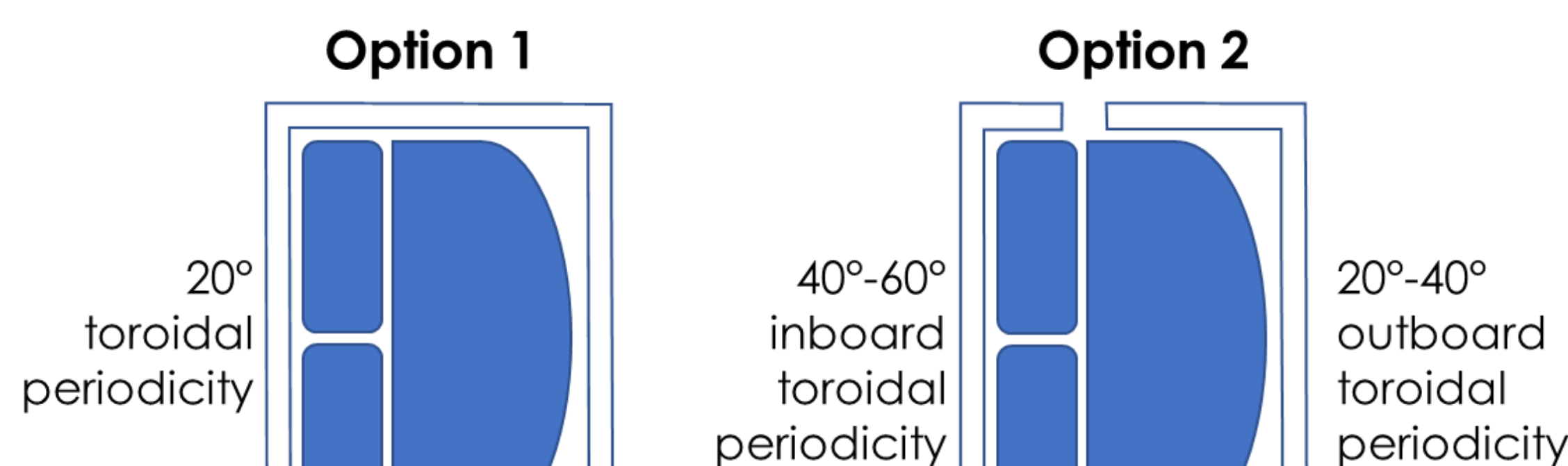
Cryostat THS

- Option 1:** active cooling + MLI
 - Gravity support: same as for VV THS
 - No contact to port THS to reduce eddy current circulation
 - Hydraulic connection: parallel to the VV and port THS
- Option 2:** use of MLI only

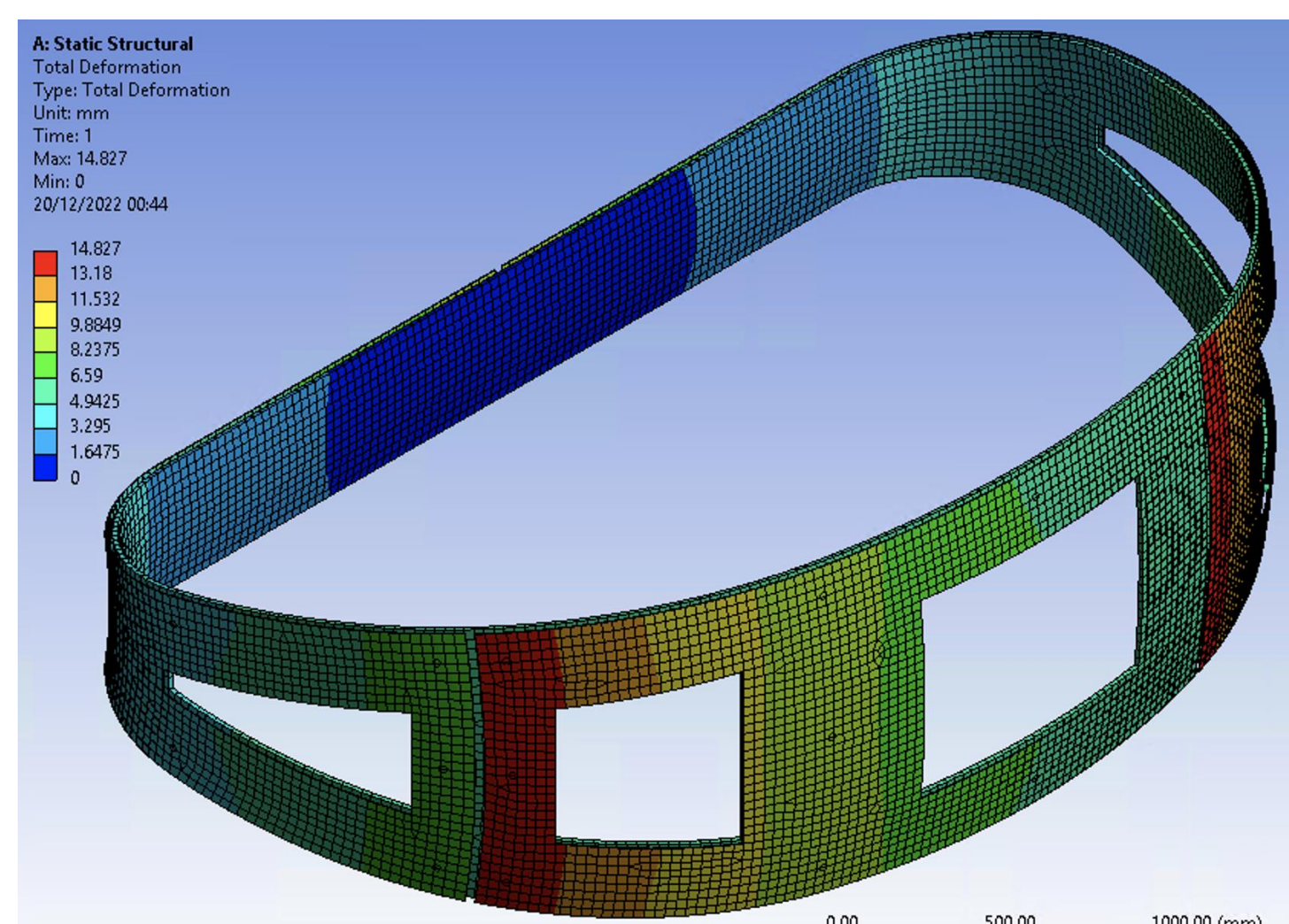
ITER CSM#	Upper shield	Scaled radiative heat losses
1	Actively cooled by LN ₂	1
2	MLI only	0.6

[Martovetsky, IEEE TPS, 2022]

Vacuum Vessel THS

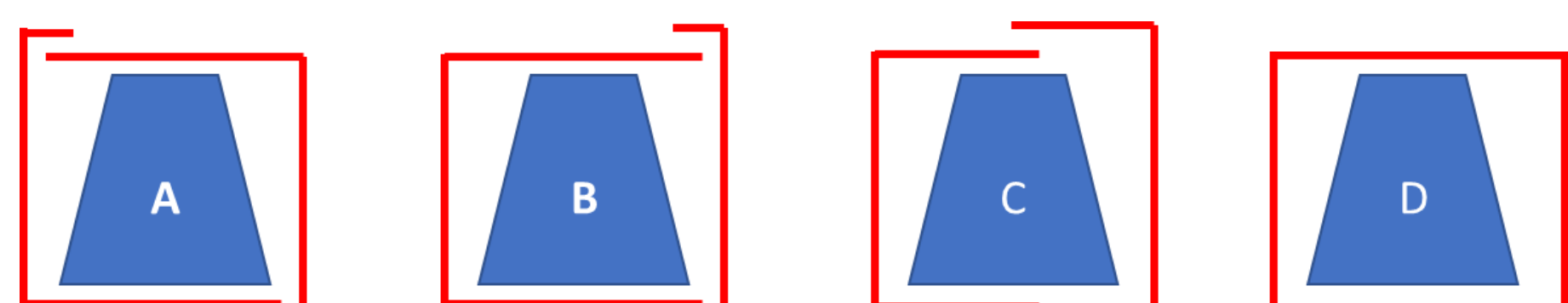
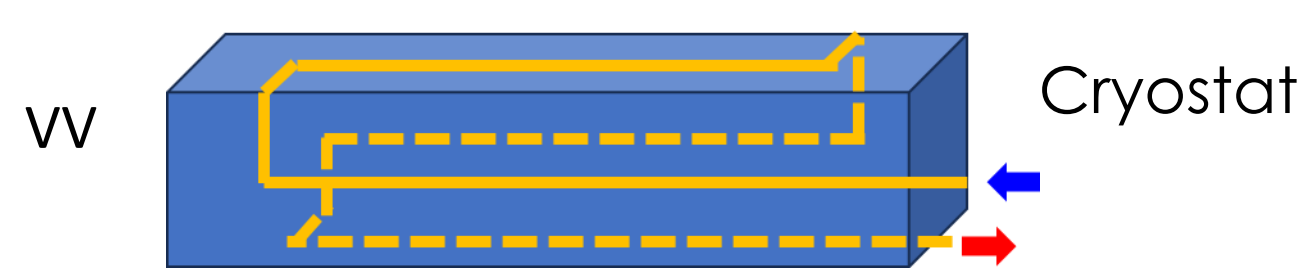


- Split poloidally the THS in 5 segments fixed to the vacuum vessel with 45 supports
- relative thermal displacements at the support points during baking:
 - ~2 mm in toroidal direction
 - ~20 mm in poloidal direction



Port THS

- Gravity support: same as for VV THS
- No contact VV-port-Cryostat THS to reduce eddy current circulation
- Hydraulic connection:
 - Parallel to the VV THS, inlet from the cryostat side
 - Series connection of the ports in the same 20° sector



Conclusions & perspective

- Status of the ongoing design of the vacuum vessel THS and conceptual design of the port and cryostat THS reported
 - Minimization of the radiative heat transfer by silver plating and active cooling
 - ITER-like (pipe welding) and clamps + copper mesh preferred solutions for pipe attachment to panels
 - Gravity supports: G10 bolts on VV, port and cryostat
- PERSPECTIVE:** finalization of the THS design, completion of the electro-magnetic, thermal-hydraulic and mechanical analyses (pressure drop, seismic loads, ...) and down-selection of the best options

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