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(Article begins on next page)
Prediction, experimental results and analysis of the ITER TF Insert Coil quench propagation tests, using the 4C code

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Abstract – The ITER Toroidal Field Insert (TFI) coil is a single-layer Nb$_3$Sn solenoid tested in 2016-2017 at the National Institutes for Quantum and Radiological Science and Technology (former JAEA) in Naka, Japan. The TFI, the last in a series of ITER insert coils, was tested in operating conditions relevant for the actual ITER TF coils, inserting it in the borehole of the Central Solenoid Model Coil, which provided the background magnetic field. In this paper, we consider the five quench propagation tests that were performed using one or two inductive heaters (IHs) as drivers; out of these, three used just one IH but increasing delay times, up to 7.5 s, between the quench detection and the TFI current dump. The results of the 4C code prediction of the quench propagation up to the current dump are presented first, based on simulations performed before the tests. We then describe the experimental results, showing good reproducibility. Finally, we compare the 4C code predictions with the measurements, confirming the 4C code capability to accurately predict the quench propagation, the evolution of total and local voltages, as well as of the hot spot temperature. To the best of our knowledge, such a predictive validation exercise is performed here for the first time for the quench of a Nb$_3$Sn coil. Discrepancies between prediction and measurement are found in the evolution of the jacket temperatures, in the He pressurization and quench acceleration in the late phase of the transient before the dump, as well as in the early evolution of the inlet and outlet He mass flow rate. Based on the lessons learned in the predictive exercise, the model is then modified to try and improve a posteriori (i.e. in interpretive, as opposed to predictive mode) the agreement between simulation and experiment.

(Some figures may appear in colour only in the online journal)

I. INTRODUCTION

The safe application of any code to predictive (or design) exercises concerning forthcoming or future superconducting (SC) fusion machines, like ITER, JT-60SA, or DEMO, should in principle be preceded by, and rely on, its successful validation, i.e. on a good agreement between simulations and measurements, aimed at confirming that the code implements the right model and that the model equations are correctly solved. Furthermore, separate validation exercises should be conducted for different phenomena (e.g., stability, AC losses, quench, cooldown, etc), in view of the different physics and time scales involved. Also, it should be important to distinguish between validation exercises where the simulations are performed after/with/taking advantage of the knowledge of the results of the measurements, and rigorously predictive (i.e. blind) validation, where the simulations are performed before/without the knowledge of the results of the measurements, which are obviously much harder. Indeed, code validation, with special reference to predictive validation, is considered a very important and needed step in different big communities, e.g., the American Institute of Aeronautics and Astronautics (AIAA), the American Nuclear Society (ANS) or the American Society of Mechanical Engineers (ASME), where the question of the verification and validation of computational modeling has been developed in the last few decades to a level of unprecedented maturity and rigor, see for instance [1], [2].

Quench propagation in cable-in-conduit conductors (CICCs) or magnets for nuclear fusion has attracted the interest of the superconductor science and technology community for many years. After the first studies in the ‘80s [3], the QUench Experiment on Long Length (QUELL) was carried out mid of the ‘90s on an ITER-like (albeit sub-size) CICC, characterized by the presence of a central cooling channel [4], [5], [6]. In the first
decade of this century, the major role in increasing the database related to this fundamental phenomenon was played by the tests of the ITER Insert Coils [7], [8], [9] [10], [11], which gave rise to a series of computational studies specifically devoted to the topic, [12], [13], [14], [15], [16], [17].

However, notwithstanding the non-negligible extent of the quench-related database today, the validation of the existing thermal-hydraulic (TH) models and computational tools for this fundamental transient in SC magnets is still far from being a routine step, and if we talk about predictive validation, no such exercise can be found in the literature, to the best of our knowledge.

With the above-mentioned background in mind, and following the roadmap proposed in [18] to confirm the reliability of the existing TH codes for SC magnets, we have taken advantage of the (then) forthcoming second ITER Toroidal Field (TF) Insert (TFI) [19] coil test, to try and confirm the predictive capabilities of the state-of-the-art 4C code [20]: a set of simulations of quench propagation in the TFI was performed, as adherent as possible in input to the TFI test program but still before the test. As a proof of the blind nature of these simulations, selected results thereof were distributed among the members of the testing group before the tests were actually performed.

The paper is organized as follows: we first introduce the relevant experimental setup and available diagnostics in the TFI, which were of course the fundamental input of the simulations; then we present the predictive 4C quench model and a selection of its results; then we describe the experimental results of all the quench tests performed in the TFI; then we compare the predictions with the measurements; finally, we introduce an upgrade of the 4C model, developed on the basis of the lessons learned in the predictive exercise, and show how this improves the interpretive capabilities of the 4C code as far as the TFI quench propagation is concerned.

II. EXPERIMENTAL SETUP

The TFI coil, see Fig. 1, is a 43 m long single-layer solenoid wound in the grooves of a stainless-steel (SS) mandrel, using one of the Nb$_3$Sn circular CICC (see Fig 2) adopted for the actual ITER TF coils and cooled with supercritical He in forced circulation at 4.5 K. The TFI is so far the last item in the above-mentioned series of ITER Insert Coils [7], [8], [9], [10], [11], all tested in the bore of the ITER Central Solenoid Model Coil (CSMC) [21] at Naka, Japan, in conditions relevant for the actual ITER operation. Table 1 and Table 2 report the main geometrical and scaling parameters [24] for the TFI conductor.

![Fig. 1. (a) Section of the TFI assembly and (b) picture of the TFI before installation in the bore of the CSMC (courtesy of QST).](image)

![Fig. 2. Cross section of the TFI conductor.](image)
Several tests were devoted to the study of quench propagation, aimed at the assessment of the hot spot in the conductor. The quench was initiated pulsing an inductive heater (IH), wrapped around the conductor at mid length of the TFI, at increasing energies. Different delay times (3 s, 5 s, 7.5 s) after quench detection were imposed, before the 68 kA TFI current was dumped on an external resistor.

The TFI coil is well equipped with a set of diagnostics that includes thermometers, pressure taps and flow meters at the insert inlet and outlet, as well as several voltage taps and Cernox thermometers along its length, see Fig. 3.

Several tests were planned (and eventually performed) in the expected operating conditions for the ITER TF, i.e. transport current $I_{TFI} = 68 \text{kA}$ and peak magnetic field $B_{peak} = 11.8 \text{T}$. As in all previous ITER Insert Coil tests, the quench was planned to be initiated pulsing for 40 ms the inductive heater IH02, approximately located at the central turn of the conductor. (Eventually, also a single test simultaneously pulsing two IHs was performed.) This explains the non-uniform distribution of the diagnostics along the coil length, i.e. their refinement close to the IH region.

The TFI quench tests were planned (and eventually performed) in the expected operating conditions for the ITER TF, i.e. transport current $I_{TFI} = 68 \text{kA}$ and peak magnetic field $B_{peak} = 11.8 \text{T}$. As in all previous ITER Insert Coil tests, the quench was planned to be initiated pulsing for 40 ms the inductive heater IH02, approximately located at the central turn of the conductor. (Eventually, also a single test simultaneously pulsing two IHs was performed.) This explains the non-uniform distribution of the diagnostics along the coil length, i.e. their refinement close to the IH region.

Fig. 3. Sketch of the available diagnostics installed along the coil length [23]: VTxx and starxx voltage taps, TSxx (Cernox, on the jacket) and TCxx (Carbon, on the He pipes) thermometers, PT-xx pressure taps and PCT-xx flow meters. IH02 indicates the inductive heater used for triggering most of the quench tests. The thick red arrows show the direction of the He flow.

Fig. 4. Graphical representation of the quench timeline, with TFI and CSMC current dumps.

Table 1. Main geometrical parameters for the TFI conductor [22], [23].

<table>
<thead>
<tr>
<th>Parameter</th>
<th>Value</th>
</tr>
</thead>
<tbody>
<tr>
<td>Jacket external diameter [mm]</td>
<td>43.83</td>
</tr>
<tr>
<td>Jacket internal diameter [mm]</td>
<td>39.8</td>
</tr>
<tr>
<td>Void fraction in the bundle region</td>
<td>31.3%</td>
</tr>
<tr>
<td>Central channel ID/OD [mm]</td>
<td>8/10</td>
</tr>
<tr>
<td>Total He cross section [mm$^2$]</td>
<td>420</td>
</tr>
<tr>
<td>Number of strands (SC + Cu)</td>
<td>900+522</td>
</tr>
<tr>
<td>Strand diameter (SC / Cu) [mm]</td>
<td>0.822 / 0.821</td>
</tr>
<tr>
<td>$\cos(\theta)$</td>
<td>0.9699</td>
</tr>
<tr>
<td>Wrapping area [mm$^3$]</td>
<td>34</td>
</tr>
<tr>
<td>Cabling pattern</td>
<td>(2 SC + 1 Cu) × 3 × 5 × 5 + (3 Cu × 4 Cu) × 6</td>
</tr>
</tbody>
</table>

Table 2. ITER style $I_c (B, T, \epsilon)$ parameterization for the TFI conductor [23].

<table>
<thead>
<tr>
<th>Parameter</th>
<th>Value</th>
</tr>
</thead>
<tbody>
<tr>
<td>$C_{[A \times T]}$</td>
<td>33$\times 10^3$</td>
</tr>
<tr>
<td>$B_{c,20m} [T]$</td>
<td>32.35</td>
</tr>
<tr>
<td>$T_{m0} [K]$</td>
<td>16.22</td>
</tr>
<tr>
<td>$p [-]$</td>
<td>0.84</td>
</tr>
<tr>
<td>$q [-]$</td>
<td>2.57</td>
</tr>
<tr>
<td>$C_{a1} [-]$</td>
<td>47.02</td>
</tr>
<tr>
<td>$C_{a2} [-]$</td>
<td>11.76</td>
</tr>
<tr>
<td>$\epsilon_0 [-]$</td>
<td>2.31$\times 10^{-3}$</td>
</tr>
<tr>
<td>$\epsilon [-]$</td>
<td>3.97$\times 10^{-3}$</td>
</tr>
</tbody>
</table>
The definition of quench detection (QD) is based on the value of the total voltage (ITF-VD-ALL) measured across the TFI. While the coil is in operation, at any time \( t \) the ITF-VD-ALL signal is integrated along the conductor over the previous 0.1 s by the QD system: if the result is larger than the prescribed threshold of 0.01 V \( \times \) s, a quench is detected, see Fig. 4. In that case, the quench freely propagates for a prescribed delay time \( t_{\text{delay}} \), after which a signal triggers the dump of the TFI current onto a dump resistor, by opening the coil electric circuit. (About 0.45 s after the TFI dump, also the CSMC current is dumped to avoid additional heat deposition in the TFI because of combined AC losses, but we will not consider this phase in the paper, as all of our simulations will stop at the dump of the TFI).

A set of delay times was planned in the test program, increasing up to a maximum \( t_{\text{delay}}^{\text{max}} = 7.5 \) s. This delay time was chosen in order to assess the TF conductor behavior after a quench, featuring a Joule energy deposition as similar as possible to that of a detected quench of a TF coil in ITER. According to the ITER design criteria [25], the quench detection and switch opening will take \( \tau_{\text{det}} = 2 \) s, during which the coil will still operate at full current \( I_0 \). In this phase, the energy deposition \( E_{j,\text{det}} \) is

\[
E_{j,\text{det}} = R I_0^2 \tau_{\text{det}}
\]  
(1)

then the current will undergo a fast discharge

\[
I(t) = I_0 \exp(-t/\tau_{\text{dump}})
\]  
(2)

with a time constant \( \tau_{\text{dump}} = 11 \) s. The energy \( E_{j,\text{dump}} \) deposited during this exponential decay, until the coil has been fully discharged, and under the assumption that the resistance is constant, is

\[
E_{j,\text{dump}} = R \int_{t_{\text{dump}}}^{\infty} (I(t))^2 dt = R I_0^2 \frac{\tau_{\text{dump}}^2}{2}
\]  
(3)

so that the total energy deposited in an ITER TF coil during a quench is

\[
E_j = E_{j,\text{det}} + E_{j,\text{dump}} = R I_0^2 \left( \tau_{\text{det}} + \frac{\tau_{\text{dump}}^2}{2} \right)
\]  
(4)

As in the TFI

\[
E_j = R I_0^2 \left( t_{\text{delay}} + \frac{\tau_{\text{dump}}^2}{2} \right)
\]  
(5)

and \( \tau_{\text{dump}} \) is negligibly small (\( -0.1 \) s) with respect to \( t_{\text{delay}} \),

\[
t_{\text{delay}}^{\text{max}} = \tau_{\text{det}} + \tau_{\text{dump}} / 2.
\]

### III. QUENCH PREDICTIONS

The 4C code has been used for all the simulations presented in this paper.

The previous validation and application history of the 4C code covers a large set of experiments, including the two ITER Model Coils [26], [27], for which, however, no quench tests were ever performed, and the above-mentioned Insert Coils. Very different types of transients were considered, including the quench, but always in an interpretive form (i.e., after the tests were performed).

In fact, for the 4C code we can only claim so far only one strictly predictive (i.e. blind) validation test, namely the study of different types of pulsed heat load smoothing scenarios based on the use of the HELIOS SHe loop [28]. To the best of our knowledge, that remains at this time the only strictly predictive (i.e. blind) validation test of any TH code for superconducting fusion magnets.

#### A) Simulation setup

For the TFI quench prediction we focused on the case \( t_{\text{delay}} = t_{\text{delay}}^{\text{max}} \) which obviously includes the quenches at shorter \( t_{\text{delay}} \) as the quench is computationally reproducible by definition (i.e., the first, e.g., 3 s of the propagation of a 7.5 s delay quench will automatically coincide with the whole propagation of a 3 s delay quench in the simulation).

The parameters which must be set in input include:

- The identification of the computational domain and the respective boundary conditions, if needed → we simulate the entire conductor, including the joint region, and we impose fixed pressure at the TFI inlet and outlet, \( p_{\text{in}} \approx 5.6 \) bar, \( p_{\text{out}} \approx 5.5 \) bar for the sake of simplicity. We impose also a fixed inlet temperature \( T_{\text{in}} = 5.7 \) K, as long as no backflow occurs at the coil inlet;
- The magnetic field map \( B(x) \) and hoop strain distribution along the conductor → we use the maps shown in Fig. 5 [29]; note that the total strain is obtained summing up the contribution of the hoop strain, the thermal strain (\( -0.7\%) \) and the extra strain (\( -0.04\%) \), calibrated by means of the analysis of current sharing temperature \( (T_{\text{CS}}) \) measurements in the TFI, as recently done for the Central Solenoid Insert Coil (CSIC) [30];
- The value of the conductor \( n \)-index \( = 5 \) for the superconducting-to-normal state transition region, calibrated by means of the analysis of the TFI \( T_{\text{CS}} \) measurements;
The initial condition: imposed linear pressure drop between $p_{in}$ and $p_{out}$ (resulting in an initial mass flow rate of $\sim 8$ g/s) and temperature profile obtained running a simulation with prescribed $T_{in}$ until a steady-state is reached without power input from the IH;

The linear power density $q'(x,t)$ to be applied to the conductor and its repartition between cable and jacket, to properly mimic the power deposition by the IH $\Rightarrow$ this is in principle a delicate choice, but experience from the simulation of many quenches in other ITER Insert Coils teaches that, as opposed to the quench initiation, quench propagation is not significantly affected by these parameters. Therefore, we chose a sufficiently high energy to get a quench propagation in the system ($\sim 110$ J) and deposited it uniformly both in time (40 ms duration) and in space along the conductor (112 mm, coinciding with the IH length). 80% of the power was deposited in the jacket and the rest in the cable;

The constitutive relations for the 1D conductor model, and namely the friction factors for the cable bundle and for the central channel region, the heat transfer coefficients (HTCs) between the different conductor constituents and the HTC between conductor and mandrel, which were found best-fitting with the 4C code another set of pure thermal-hydraulic (i.e., no current) independent TFI data [31];

From the numerical point of view, adaptive time-step (from 0.5 ms to 100 ms) and mesh size (from 0.25 mm to 100 mm) have been adopted as a result of a dedicated convergence study.

B) Simulation results

We present in this section a selection of the results of the predictive simulations, aimed at giving to the reader an idea of the wealth and level of detail of the information which can be extracted from a code run. Moreover, they focus on the evolution of some of the quantities for which a direct comparison with the measurements will be presented below. These quantities are: the local voltages $V_{loc}$ across pairs of neighboring voltage taps close to the IH; the maximum strand temperature; the maximum pressurization in the conductor, localized under the IH02, with respect to the instant ($t = 0$) when the IH is switched on; the inlet and outlet He mass flow rates $dm/dt$. All of the evolutions are reported in these figures as functions of $t^* = t - t_{QD}$

whereas the initial time in the plots corresponds to $t = 0$ s.

Fig. 6. Predicted evolution of the local voltages.

Fig. 6 clearly shows the downstream and upstream propagation of the quench (progressive takeoff of the voltage signals downstream and upstream of the IH, respectively).

The knees on the local voltage traces, for instance at $t^* \sim 0$ s on ITF-VD-1112g, or at $t^* \sim 2$ s on ITF-VD-1011g and ITF-VD-1213g, are due to the fact that, at those times, the whole length between the respective taps has become normal. The further voltage increase becomes then slower, as it is only due to the temperature increase and related increase of the Cu resistivity.

While from these data the evolution of the total computed voltage across the coil can be easily deduced (shown below in the comparison with the experiment), they are used here to deduce the propagation of the quench front, represented in the $(x,t)$ characteristics plane in Fig. 7. The passage of the quench front beyond a voltage tap is assumed to occur when the voltage
measured between that voltage tap and its neighbor ahead of the quench front reaches the conventional threshold of 1 mV. This is done in order to perform a fair comparison with the experimental data, for which the same voltage threshold is adopted to detect the entrance of the quench front in the space between two voltage taps, see below. In addition, the set \( x_0(t) \) of the points where \( I = I_C \) is also reported, as it better identifies the quench front propagation in the simulation. The approximate quench propagation speed \( V_q \) between two neighboring voltage taps can then be easily deduced from the time needed to propagate from one voltage tap to the neighboring one, and is also reported in Fig. 7, showing what appears to be a rather ubiquitous feature of the quench in ITER CICCs, namely its progressive acceleration [32].

![Fig. 7. Predicted normal zone propagation with 7.5 s delay in the characteristic \((x, t)\) plane. Symbols: passage of the quench front beyond a voltage tap, assumed to occur when the local voltage overcomes 1 mV; dotted lines: trajectory of the points where \( I = I_c \).](image)

Just as in the case of the recent quench test of the CS Insert Coil [17], the model can be used to confirm that also in the TFI the quench is a classical pressure-driven quench, propagating with the area weighted average of the He flow speed in the two CICC regions at the quench front, see Fig. 8a. The preheating of the cable ahead of the quench front originates a broader and broader region with relatively low margin ahead of the quench front as time goes by, as reported in Fig. 8b, which favors the quench acceleration.

![Fig. 8. (a) Predicted area-weighted average He speed distribution in the TFI central zone during quench propagation at different times \( t^* \). (b) Predicted distribution of the temperature margin in the TFI central turns during quench propagation at different times \( t^* \).](image)

The local voltage signal across the IH (ITF-VD-1112g in our case), shown in Fig. 6, is also routinely used, together with the temperature dependence of the Cu resistivity, to experimentally estimate the hot spot temperature \( T_{HS} \) — the so-called virtual thermometer, see Fig. 9. Assuming that the cable temperature is uniform between the voltage taps \( VT_{11} \) and \( VT_{12} \) embracing IH02, i.e. \( T_{1112} \), the temperature of the cable in the normal zone can be extracted from the measured \( VD_{1112} = VT_{12} - VT_{11} \), following a well-known procedure [12]. [15]. Solving for \( T_{1112} = T_{hot}\) spot the implicit equation (7):

\[
VD_{1112} = \rho_{Cu}(T_{1112}) \times (L_{1112}/A_{Cu}) \times I_{TFI} \quad (7)
\]

where \( \rho_{Cu} \) is the copper electrical resistivity, \( L_{1112} \) is the strand length between \( VT_{11} \) and \( VT_{12} \), \( A_{Cu} \) is the total copper cross section and assuming, for sake of simplicity and conservative reasons, that all the current flows in the Cu.

The maximum strand temperature, also shown in Fig. 9, is obviously always above the virtual thermometer.
signal, since the strand temperature profile across ITF-VD-1112g will not be uniform but peaked. On the other hand, the difference between the two is not that big in this case, thanks also to the small distance (1.132 m) between VT11 and VT12.

The coil starts pressurizing during the inductive pulse, see Fig. 9, and on the sound timescale also the mass flow rate at the coil boundaries reacts, leading to backflow at the inlet, see Fig. 11.

Then the pressure decays rather quickly to the initial value, until the Joule heating due to the formation of the normal zone takes off. At that time the pressure rises again very quickly, until again on the sound timescale a strong expulsion of helium, both at the coil inlet and outlet, see Fig. 11, leads to a second slower pressurization phase.

Table 3 reports a summary of the IH-driven quench tests performed on the TFI, with distinguishing features in boldface.

<table>
<thead>
<tr>
<th>Delay [s]</th>
<th>Shot #</th>
<th>IH</th>
<th><code>Vdc</code> [V]</th>
<th><code>∫I^2dt</code> [A^2s]</th>
</tr>
</thead>
<tbody>
<tr>
<td>3</td>
<td>109 – 11</td>
<td>IH02</td>
<td>80.0</td>
<td>640</td>
</tr>
<tr>
<td>5</td>
<td>110 – 6</td>
<td>IH02</td>
<td>80.0</td>
<td>652</td>
</tr>
<tr>
<td>7.5</td>
<td>113 – 10</td>
<td>IH02</td>
<td>82.7</td>
<td>676</td>
</tr>
<tr>
<td>3</td>
<td>118 – 8</td>
<td>IH02 + IH01</td>
<td>132.2</td>
<td>711</td>
</tr>
<tr>
<td>3</td>
<td>120 – 5</td>
<td>IH02</td>
<td>90.5</td>
<td>806</td>
</tr>
</tbody>
</table>

The test results have a nice reproducibility for what concerns both the global (`Vma`) and the local voltages, see Fig. 12. (Note that the quench detection time used in the definition of `t*` is now the experimental one, i.e. `t*` here and in the previous section are not exactly coincident.)

The normal zone obviously initiates between VT11 and VT12, as the IH is located there.

The flattening of the experimental trace of the local voltage ITF-VD-1112g for `t* > ~6.5` s after the quench detection is due to the saturation of the sensor.

The quench front propagation, deduced as explained above from the local voltage signals take-off, is reported in Fig. 13 for all the IH-driven quench tests, showing again excellent reproducibility. The normal zone propagation is almost symmetric with respect to the IH location and just a little faster in the downstream direction, because of the background (initial) He flow. The quench fronts progressively accelerate both upstream
and downstream, from an initial speed of ~0.5 m/s to more than 1.5 m/s, as estimated from the time needed to propagate from one voltage tap to the neighboring one in Fig. 13.

The hot spot temperature, deduced from the local voltage signals as explained above, also suffers from the above-mentioned saturation, see Fig. 14. An experimental assessment of the maximum temperature reached by the conductor at the time of the dump in the quench test characterized by the longest delay time is then unfortunately possible only through extrapolation.

Fig. 12. Measured evolution of the (a) total and (b) local voltages during TF1 quench tests for shot #109 – 11 (solid), #110 – 6 (dotted with circles), #113 – 10 (dash-dotted), #118 – 8 (dashed) and #120 – 5 (solid with diamonds). In (b) different colors refer to different local sensors, while different line-styles refer to different experimental shots.

Fig. 14. Evolution of the virtual hot spot temperature deduced from the VD_{1112} measurement (see text). The trace for shot #113 – 10 ($t_{\text{delay}} = 7.5$ s) flattens at ~140 K due to the saturation of the VD-1112 voltage signal, see Fig. 12b.

Fig. 15. Measured evolution of the jacket temperature at different locations (see also Fig. 3) during the quench propagation. Different colors refer to different local sensors, while different line-styles refer to different experimental shots.

The measured jacket temperature evolution at different locations, see Fig. 15, is not as perfectly reproducible in the different quench tests as the previously considered variables, but shows a spread up to 3.5 K. Furthermore, TS03 shows a somewhat odd behavior, with a trace that does not run parallel to the others, but instead grows much more slowly, possibly as a result of a not perfect attachment of the sensor to the jacket, resulting in an additional thermal contact resistance.
The reproducibility of the results in the TFI quench tests is confirmed also for the pressurization at the coil boundaries, which substantially shows the same behavior for all the shots, see Fig. 16a, even during the quench initiation, as highlighted in the zoom reported in Fig. 16b. The maximum pressurization reaches almost 1.5 bar at the inlet, in the test with the maximum delay.

The mass flow rate shows an evolution independent of the particular quench scenario only after the quench detection or so, see Fig. 17; on the contrary, the evolution presents some differences in the quench initiation phase, especially between $t^* = -1.5$ s and $-0.5$ s, as shown in Fig. 17b, which do not seem to be related, e.g., to the amount of the energy deposited (see also Table 3).

About 1 s before the QD, the pressure at the boundaries increases very rapidly (Fig. 16b) because the Joule heating in the initiated normal zone at the center of the conductor causes a fast He expansion. This in turn causes, on the sound propagation time-scale between the normal zone and the TFI endings (~0.1 s), the He expulsion from the coil boundaries, see Fig. 17b. Then, between $t^* = 0$ s and 1 s, the pressurization slows down consistently because of the reduction of the He expulsion, due in turn to the above-mentioned pressurization at the boundaries (the He expulsion being driven by the pressure difference between the quenched zone and the coil boundaries).

The second fast pressurization phase, for $t^* > 5$ s, already noticed in the CSIC tests [17], is possibly related to the quench acceleration reported in Fig. 13; the flow meters do not show a further He expulsion increase in this phase because their signal saturates for $t^* > 3$ s (see Fig. 17a). Since the flow meter is of the orifice type, the data outside the calibrated range, which goes from zero to around 30 g/s, are not reliable. For this reason, they will not be reported anymore from Fig. 17b onwards.

In the case of quench initiation using two IHs, shot #118-8, no major differences are present with respect to the shots where only one IH was used. The only effect of using two IHs is to deposit more energy than the required MQE, but that does not affect the quench propagation.
V. COMPARISON BETWEEN 4C CODE PREDICTIONS AND MEASUREMENTS

For this comparison, only shot #113–10 will be considered, since it presents the longest delay time and, as such, it is the most interesting and severe from the quench propagation point of view. As the present work focuses on the quench propagation phase, between detection and dump, for the sake of the present comparison the results will be reported below as a function of a single time coordinate \( t^* \), i.e. the real time shifted by the respective quench detection time. (For the sake of completeness we just quote here that \( t_{QD}^{\text{pred}} \sim 1.71 \) s, whereas \( t_{QD}^{\text{exp}} \sim 1.64 \) s. This obviously implies that the predicted and measured curves are not exactly synchronized in reality.) The details of the transient before the quench detection are indeed strongly dependent on uncertain parameters like the amount of energy actually deposited by the IH and its repartition between cable and jacket, which are beyond the scope of the present paper as they are known to have a negligible influence on the quench propagation phase. A separate study of the TFI IH driven test results is ongoing, specifically devoted to the phase before the quench is initiated, and it will be presented elsewhere [33].

The predicted evolution of the total voltage across the TFI coil is almost indistinguishable from the measurement, see Fig. 18.

However, if we look in more detail at the evolution of the local voltages across pairs of neighboring voltage taps, see Fig. 19, we note that the perfect agreement in the global voltage is also partly the result of the casual compensation of small disagreements: indeed, the local predicted voltages take off a bit earlier than in the experiment, but then evolve a bit slower.

The comparison of the different hot spot predictions with the virtual thermometer, see Fig. 20, shows a good agreement between prediction and measurement, although the prediction somewhat overestimates the rate of hot spot temperature increase (\( \sim 24 \) K/s against \( \sim 19 \) K/s measured at \( t^* = 6 \) s). As the predicted max strands temperature is less than 15 K above the hot spot temperature predicted by the virtual thermometer, we can now reasonably estimate the actual maximum strand temperature adding 15 K to the extrapolated virtual thermometer measurement (\( \sim 155 \) K) just before the current dump, resulting in our best hot spot temperature estimate of \( \sim 170 \) K. The fact that the 7.5 s delay quench in the TFI was conducted to mimic the real ITER TF behavior allows considering this hot spot temperature as reasonably representative of the expected actual value after a quench in a TF coil: being it still well below 250 K, it satisfies the ITER design criteria [34].
The comparison between predicted and measured quench propagation speed shows a good agreement, with an error always below 15%, in the initial and central phase of the quench propagation, up to \( t^* \sim 5\) s, as shown in Fig. 21. On the contrary, in the last phase the predicted quench propagation speed is much smaller than measured, i.e. the quench is accelerating much more than predicted, with errors up to almost 50%. It should be noted that in previous quench studies, e.g. [12], [15], [17], an agreement like that shown in Fig. 21a was typically considered to be globally very good, but the punctual representation of the error in the quench propagation speed given in Fig. 21b conveys a somewhat different picture.

The respective symbols in Fig. 21 are located in space at the generic tap location and in time when the local voltage across two neighboring taps reaches 1 mV, see Fig. 22 for an example. It is seen from this figure that the 1 mV threshold is suitably chosen to stay just above the noise level in the measurement.
Despite the very good agreement between prediction and measurements shown above, the predicted jacket temperatures significantly overestimate the experimental results, see Fig. 23, and tend to rise earlier and faster. This level of disagreement was unexpected, since in previous quench simulations, e.g., in the case of the CSIC, the 4C code had shown a very good capability to reproduce the jacket temperature evolution [17], even if the jacket was thick and square instead of thin and circular.

One possible explanation for the discrepancy between predicted and measured jacket temperatures is that the Cernox thermometers are in reality glued on the external surface of the jacket, see Fig. 24, and the glue layer, not included in our model, delays the response of the sensor to the evolution of the jacket temperature. As a last comment on this point, similar results to those predicted with 4C (and therefore similar disagreement with the experiment) were also found after the tests in analyses performed with another computational tool, the Supermagnet code [35], by other authors [36].

The predicted mass flow rate evolution, see Fig. 25, shows two distinct phases. In the first one, just after the IH pulse, we have a complete disagreement between prediction and measurement: while, e.g., the predicted He flow reverses at the TFI inlet, as intuitively expected, the measured one increases, albeit only slightly (the opposite happens at the outlet, where instead of the predicted, and intuitive, initial increase of the flow, a reduction of the flow is recorded by the flowmeter). This feature, already noted in the last CSIC test, remains therefore unexplained. In the second phase, an anticipated and stronger expulsion of the helium at the outlet is predicted by the model, compared with the measurement. This difference is due to the fact that in the experiment the He expulsion from the coil tends to pressurize the manifolds at the coil boundaries, in turn reducing the pressure difference between the quenched zone and the coil boundaries that drives the He expulsion, see above.

VI. INTERPRETIVE 4C CODE SIMULATIONS OF THE TFI QUENCH
The focus of the interpretive simulations, performed after the test and presented in this Section, is on trying to improve the model especially considering the quantities not well reproduced by the predictive simulations. These include the jacket temperature and the quench acceleration in the last phase of the propagation before the dump as well as the inlet/outlet mass flow rates (and pressurizations).

A. Including the structures in the model

The first addition to the predictive model is the model of the structures. The rationale for this addition is that we want to check if the measured jacket temperature is at least bracketed by the computed jacket and structure temperatures.

The SS mandrel, holding the TFI, was discretized azimuthally in 8 2D “cuts” where the heat conduction problem is solved with the Finite Element method, see Fig. 26. The number of sections, as well as the number of triangles into which each section has been discretized (from 6500 to 22000 triangles, depending on the particular cut considered), have been defined based on our previous experience on similar geometries [37] and transients [38].

The structures temperature is monitored at selected locations on the 2D cuts, as indicated in Fig. 26.

As expected, it turns out that the computed jacket and structure temperatures bracket the measured jacket temperatures, see Fig. 27.

Fig. 26. Top: sketch of the structures model implemented in 4C code showing the geometry of the conductor grooves and the additional temperature monitor used in the simulations, on the 5th turn, at the boundary between the structures and the CICC, indicated by the red solid circle. The location of the mandrel temperature sensors is also reported (orange rectangles). Bottom: location of the jacket temperature sensors on the central turn with respect to the position of the cuts used to discretize the 3D structures.
The addition of the structures in the interpretive model also allows computing the mandrel temperature at the sensors ITF-TS-11L and ITF-TS-12L, located on the external surface of the SS mandrel in correspondence of the 5th TFI turn as shown in Fig. 26. The computed results are in qualitative agreement with the measured temperature evolutions, see Fig. 28. The quantitative discrepancy may be related to the fact that the FE model adopted features a finite number of 2D sections; as a result, the virtual sensors, located at the same \((r, z)\) position of the real sensors, are actually representative of a toroidal segment of the mandrel, while the temperature measurement obviously refers to a single toroidal coordinate. Indeed, the agreement in the evolution of the average mandrel temperature between the two locations is very good.

B. Including a different treatment of the boundary conditions

The second modification/extension of the predictive model, implemented for the interpretation after the tests, is to use either of the following two improved alternative recipes for the boundary conditions:

i) include an external cryogenic circuit to avoid the need of boundary conditions \([39]\) (since in principle this recipe cannot be better than the second recipe in an interpretive simulation, it is only justified by the interest to model also \(p_{\text{in}}(t), p_{\text{out}}(t), T_{\text{in}}(t)\) and compare them with the measurements);

ii) adopt the measured \(p_{\text{in}}(t), p_{\text{out}}(t), T_{\text{in}}(t)\).

According to recipe i), a simplified model of the TFI, bus bars (BBs) and CSMC cooling circuits, the latter included in the model as simple pipes, just to preserve the main features of the real circuit, has also been implemented, following a similar model adopted in \([17]\) for the CSIC, see Fig. 29, as an alternative to imposing the measured boundary conditions.
The agreement of the simulated pressurization at the TFI boundaries with the measurement, see Fig. 30, is only qualitative even for $t^* < 5$ s: the pressurization is anticipated by the code, consistently with the anticipated He expulsion shown in Fig. 31 below.

The second phase of the quench propagation (for $t^* > 5$ s), characterized as observed above by an accelerated pressurization at the TFI boundaries, is not properly captured by the simulation even in this case.

Adopting recipe ii) above, the measured He mass flow rates at the inlet and outlet of the coil are not better reproduced, compared to the 4C predictions with fixed BCs, see Fig. 25 above.

As expected [39], the computed propagation is also slower than what was found above in the predictive runs, due to the pressurization at the boundaries.

C. Parametric study of the effects of inter-turn heat transfer

In order to try and address the question of the lasting discrepancy between simulated and measured acceleration of quench and pressurization at the boundaries for $t^* > 5$ s, it may be useful to observe that it takes about that time for the quench to propagate both upstream and downstream along an entire turn, i.e., to reach again the azimuthal region where the IH is located.

Since it is well known, see e.g. Fig. 8, that preheating ahead of the quench front is the cause of quench acceleration, we conjecture that the actual inter-turn heat transfer is much more effective than what hypothesized so far. In particular, a third set of interpretive simulations was carried out, where the central turn (on which the IH is installed) is thermally coupled to the two neighboring ones directly across the $\delta_{in} = 1.4$ mm thick turn insulation, i.e. neglecting the presence of the structures in between. The contact perimeter is assumed equal to its
maximum value, i.e. half of the jacket circumference. (Clearly, although this increased coupling is assumed to act all along the three turns, it will be most effective where the temperature differences are significant, i.e., below the IH.) As this should give the maximum possible thermal coupling among the turns, the inter-turn heat transfer coefficient $HTC_{IT}$ computed across the nominal insulation thickness $2\times \delta_{ins}$ is then parametrically scaled by a factor $M_Q$ between 0 and 1 as follows

$$HTC_{IT} = M_Q \times k_{ins} / (2 \times \delta_{ins})$$  \hspace{1cm} (8)

where $k_{ins}$ is the thermal conductivity of the insulation material.

The results of the present sensitivity study are presented in Fig. 32, where it is seen that, with two suitable values of $M_Q$, it is possible to at least bracket the measured evolution of the quench propagation.

![Fig. 32](image1.png)

Fig. 32. (a) Normal zone propagation with two different multipliers $M_Q$ of the inter-turn thermal coupling. (b) Comparison of measured (blue) and computed with two values of $M_Q$ (green and orange, respectively) quench propagation speed at different locations of the quench front.

Also the accelerated pressurization for $t^* > 5$ s at both inlet and outlet is now better captured, at least qualitatively, see Fig. 33.

![Fig. 33](image2.png)

Fig. 33. Comparison between measured (thick solid black) and simulated with two values of $M_Q$ (thin dashed red and thin dash-dotted green, respectively) inlet (a) and outlet (b) pressure. All pressures are rescaled to the respective initial values.

VII. CONCLUSIONS

The results of the predictive analysis, of the experimental test performed at QST Naka, Japan, and of the interpretive analysis of quench propagation in the ITER Nb$_3$Sn Toroidal Field Insert coil have been presented and compared.

The quenches were initiated using an Inductive Heater, as customary for the ITER Insert coils, and the results of tests adopting different delay times between quench detection and TFI current dump generally showed excellent reproducibility.

All the simulations presented in the paper have been performed with the 4C code and a qualitative summary of the level of success of the code in reproducing the evolution of the single, detailed features of quench propagation is reported in Table 4, together with a similarly qualitative evaluation of the results of previous assessments of quench propagation in other ITER-relevant Nb$_3$Sn coils available in the literature.
Most of the experimental results show excellent reproducibility in the quench tests performed with different delays: in the longest, 7.5 s delay test, the global voltage across the coil increased up to \( \sim 5 \) V, the hot spot temperature increased up to \( \sim 170 \) K, and the quench accelerated from \( \sim 0.5 \) m/s to \( \sim 1-1.5 \) m/s, both upstream and downstream.

Predictions performed using the 4C code, and rigorously issued before the tests, to the best of our knowledge for the first time for an ITER-relevant quench, turned out to be in good-to-excellent agreement with the measurements for most of the relevant quantities (global and local voltages, hot spot temperature, quench propagation speed). However, the further acceleration of the quench and of the related pressurization at the coil inlet and outlet, occurring in the last third (\( \sim 2.5 \) s) of the longest test, were not captured by the predictive simulation. It should be noted, however, that especially concerning the quench propagation speed a much more severe and detailed diagnostics was introduced here, see Fig. 21b, whereas in the past, see the references considered in Table 4, an agreement like that shown in Fig. 21a would have been likely commented as globally excellent.

A major discrepancy was found in the jacket temperatures, whose dynamics was predicted to be much faster than measured. This is particularly surprising, considering that excellent agreement had been recently reported between 4C simulations and measurements, notwithstanding the much thicker jacket, in the otherwise relatively similar case of the ITER CSIC coil [17]: we speculate that this disagreement might be at least partly due to an issue of attachment of the sensors to the jacket and related contact thermal resistance. Also the dynamics of the He expulsion at the coil inlet and outlet was well predicted by 4C, except for the very initial phase before quench detection, which is likely to be very dependent on the details of the IH power deposition in the conductor and of the operating conditions at the time of the IH pulse. The very first phase of the measured mass flow rate evolution after the IH pulse contains an odd feature (\( \frac{dm}{dt} \) increasing at the inlet and decreasing at the outlet), already noticed in the CSIC quench test but not captured by the simulations and still unexplained.

The last part of the paper was devoted to the interpretive analysis of the longest delay quench, where we attempted to improve the model by including the structures (mandrel, etc.) and by either substituting the fixed values imposed as boundary conditions in the predictions with measured time dependent values, or adding a model for the external cryogenic circuit to avoid the need of boundary conditions in the first place. While these extensions allowed to include in the comparison between simulations and measurements also inlet/outlet pressures and mandrel temperatures, showing qualitative agreement, they did not lead to noticeable improvement in the features where a significant disagreement had been found as discussed above.

To better capture the further acceleration of the quench and of the related pressurization at the coil inlet and outlet, occurring in the last third (\( \sim 2.5 \) s) of the longest test, the effect of an enhanced inter-turn thermal coupling, with respect to that assumed in the predictive simulations and then modeled through the structures module in the interpretive simulations, was parametrically studied. This allowed the bracketing of the acceleration of both the quench and the pressurization measured in the last phase of the longest quench test (7.5 s delay).
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REFERENCES


[34] ITER design description document (DDD) 11-7 magnets, conductors 2009 ITER_D_2NBXXY v1.2.